

## Steady-State Hollow Electron Temperature Profiles in the Rijnhuizen Tokamak Project

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In the Rijnhuizen Tokamak Project steady-state hollow electron temperature ( $T_e$ ) profiles have been sustained with strong off-axis electron cyclotron heating, creating a region of reversed magnetic shear. In this region the effective electron thermal diffusivity ( $\chi_e^{pb}$ ) is close to neoclassical in high density plasmas. For medium density,  $\chi_e^{pb}$  is lower than neoclassical and may even be negative, indicating that off-diagonal elements in the transport matrix drive an electron heat flux up the  $T_e$  gradient.

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The transport of energy and particles across the magnetic field in a magnetized plasma is a largely unexplained phenomenon. In a tokamak, the leading candidate for a fusion reactor based on magnetic confinement, the transport problem is in first order one dimensional, due to its toroidal geometry of nested flux surfaces. All physical parameters are one-dimensional functions (often called profiles) of the normalized minor radius  $\rho$ .

A common observation in tokamak plasmas is that the electron and ion temperature profiles  $T_e(\rho)$  and  $T_i(\rho)$  always have roughly the same shape, regardless of the method and localization of the heating [1–5]. This phenomenon is usually called profile resilience; various theoretical models have been developed to explain it [6–9]. The most pronounced example of profile resilience is the peaking of the  $T_e$  profile which was found in medium density DIII-D plasmas even when strong electron cyclotron heating (ECH) was applied with off-axis absorption [10].

This Letter presents recent experimental results, showing that profile resilience can be broken by strong off-axis ECH in a medium to high density plasma. Steady-state hollow  $T_e$  profiles have been obtained for a wide range of the electron density  $n_e$  and power deposition radius  $\rho_{\text{dep}}$ . In these cases a region of negative magnetic shear exists inside  $\rho_{\text{dep}}$  after full current adaptation.

The experiments were carried out at the Rijnhuizen Tokamak Project (RTP). This machine (major radius  $R_0 = 0.72$  m, minor radius  $a = 0.164$  m, toroidal magnetic field  $B_T \leq 2.4$  T, plasma current  $I_p \leq 150$  kA, pulse duration  $\leq 600$  ms) is equipped with an ECH system [11], comprising of, amongst others, a 110 GHz, 500 kW, 200 ms gyrotron, injecting from the low field side (LFS) in second harmonic X mode. Crucial for the experiments reported here is the Thomson scattering (TS) system, measuring  $T_e$  and  $n_e$  along a vertical chord through the center of the plasma at  $\sim 110$  radial positions simultaneously, with a spatial resolution of 2.5 mm [12,13].

In a series of off-axis heating experiments with the 110 GHz gyrotron, 350–400 kW was injected into ohmic target plasmas [hydrogen,  $I_p = 80$ –120 kA,  $B_T = 1.7$ –2.4 T,

edge safety factor  $q_a = 3.3$ –5.5, central electron density  $n_e(0) = (2 - 8) \times 10^{19} \text{ m}^{-3}$ , and effective ion charge  $Z_{\text{eff}} \approx 1.5$ –2.5]. The power was deposited either on the LFS or on the high field side (HFS), at  $\rho_{\text{dep}} = 0.3$ –0.6. Profiles of  $T_e$  and  $n_e$  were taken 80 ms after the switch-on of ECH, i.e., when a steady state has been reached, taking current diffusion into account.

In low plasma current plasmas ( $I_p = 80$  kA, i.e.,  $q_a = 4.0$ –5.5), the following observations were made. At low density off-axis ECH creates flat  $T_e$  profiles inside  $\rho_{\text{dep}}$ . Above a threshold central density  $n_e(0)_{\text{thresh}} \approx 3.5 \times 10^{19} \text{ m}^{-3}$ , off-axis ECH creates steady-state hollow  $T_e$  profiles. The central hole in  $T_e(\rho)$  is attributed to the heat transfer from electrons to ions ( $p_{e-i}$ ), which increases with increasing  $n_e$ . These observations were made for both HFS and LFS absorption. Figure 1 shows the different effects of on- and off-axis heating. Figure 2 shows the transition from flat to hollow  $T_e$  with increasing density. Figure 3 shows  $T_e$  profiles for different  $\rho_{\text{dep}}$ . The  $n_e$  profile modifies slightly during off-axis heating. The electron pressure ( $p_e$ ) profile

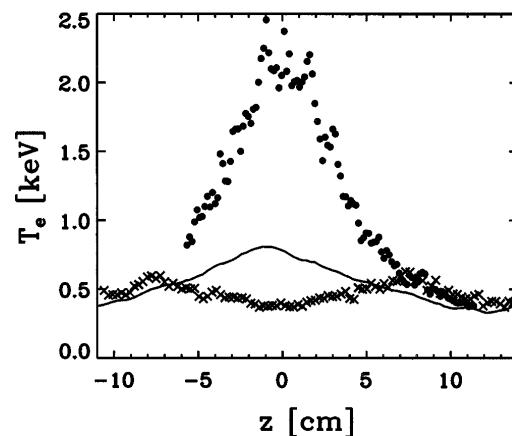


FIG. 1.  $T_e$  profiles of a centrally heated ( $\bullet$ ) and an off-axis heated ( $\times$ ) RTP discharge. For comparison the profile of a similar Ohmic discharge is plotted (full line). For these discharges  $I_p = 80$  kA and  $n_e(0) = 4.0 \times 10^{19} \text{ m}^{-3}$ .

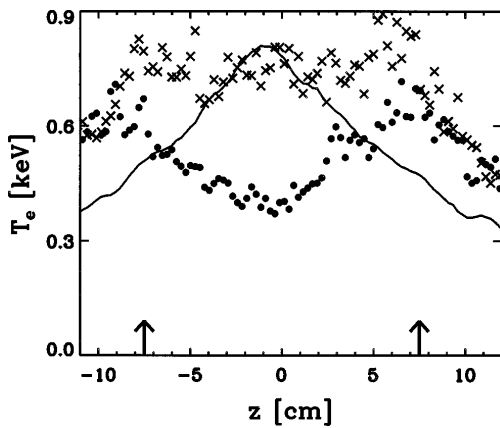


FIG. 2.  $T_e$  profiles with off-axis ECH, for  $n_e(0) = 3.0 \times 10^{19}$  ( $\times$ ) and  $4.0 \times 10^{19} \text{ m}^{-3}$  ( $\bullet$ ), showing the transition from flat to hollow  $T_e$  with increasing density (RTP discharges r19950620.008 and r19950620.019, respectively). The arrow marks  $\rho_{\text{dep}}$ . For comparison the profile of a similar ohmic discharge is plotted (full line).

flattens strongly, and can be completely flat out to  $\rho_{\text{dep}}$  under these conditions. Figure 4 shows  $T_e$ ,  $n_e$ , and  $p_e$  profiles for different  $\rho_{\text{dep}}$ . The TS profiles are confirmed by electron cyclotron emission (ECE) measurements, showing cooling in the center after the switch-on of ECH at a time scale of  $\approx 20$  ms (Fig. 5), consistent with the calculated current diffusion time. First polarimetry measurements confirmed that the current density profile ( $j$ ) adapts on this time scale [14].

At higher  $I_p$ , off-axis ECH causes a flattening of  $T_e$  out to  $\rho_{\text{dep}}$ , but no hollow  $T_e$  profiles have been observed. This is attributed to the stronger residual central Ohmic heating at high  $I_p$ , which compensates  $p_{e-i}$ .

When equilibrium has been reached, the profile of  $j$  can be deduced from the  $T_e$  profile. The hollow  $T_e$  profiles result in hollow  $j$  profiles, with reversed magnetic shear out to  $\rho_{\text{dep}}$ , and a central safety factor ( $q_0$ ) of 2.5–7.

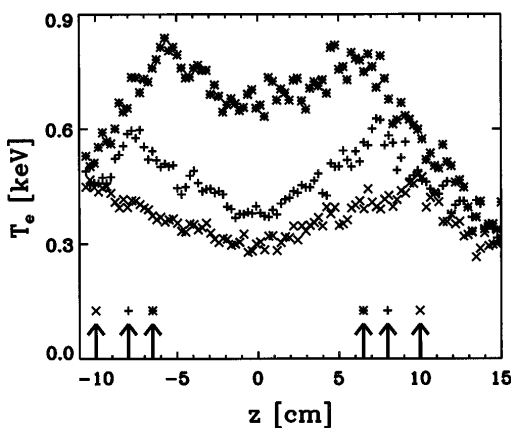


FIG. 3.  $T_e$  profiles during off-axis ECH, all with  $n_e(0) \sim 4.0 \times 10^{19} \text{ m}^{-3}$ , with different HFS absorption positions  $\rho_{\text{dep}}$ , marked by arrows [RTP discharges r19950622.053, (\*), r19950622.057 (+), and r19950622.065 ( $\times$ )].

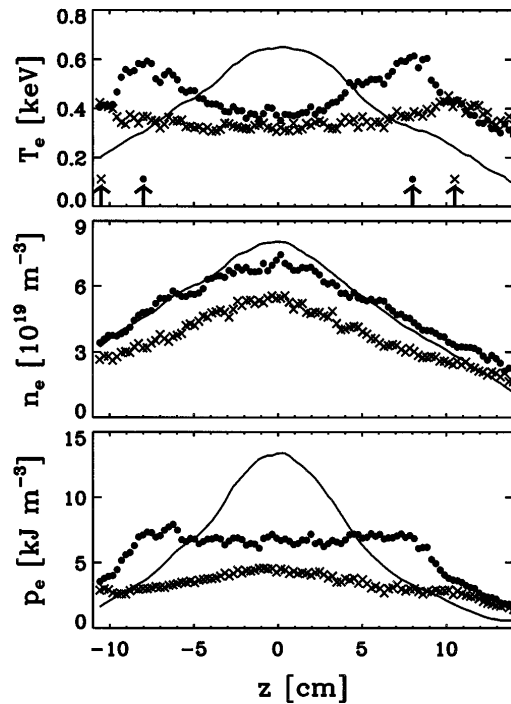


FIG. 4.  $T_e$ ,  $n_e$ , and  $p_e$  profiles during off-axis ECH with LFS absorption [RTP discharges r19950620.021 ( $\times$ ) and r19950620.026 ( $\bullet$ )]. The arrows mark  $\rho_{\text{dep}}$ . For comparison the profiles of a similar Ohmic discharge are shown (full line).

This is corroborated by polarimetry measurements, which show an increase of  $q_0$  by a factor of  $\sim 4$  compared to the Ohmically heated plasma. Further confirmation comes from the sawtooth instability, which disappears within a few ms after the heating starts. The off-axis minimum of  $q$  ( $q_{\text{min}}$ ) varies between 1.5 and 3.5. Both  $q_0$  and  $q_{\text{min}}$  increase with increasing  $\rho_{\text{dep}}$ . See Fig. 6 for a typical example.

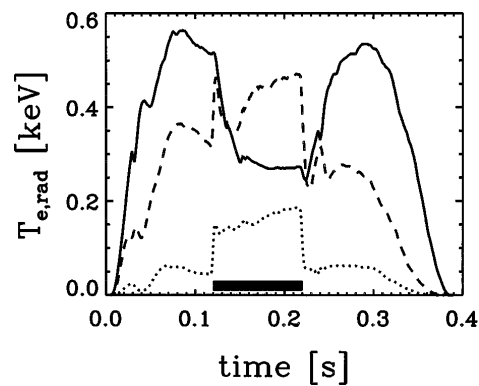


FIG. 5. Radiation temperature  $T_{e,\text{rad}}$  for discharge r19950620.021 (see Fig. 4), measured by ECE at three different positions in the plasma: in the center (full line), near  $\rho_{\text{dep}}$  (dashed line), and outside  $\rho_{\text{dep}}$  (dotted line). ECH was applied from 120 to 220 ms, as indicated by the shaded area. Note the cooling in the center after the switch-on of the heating.

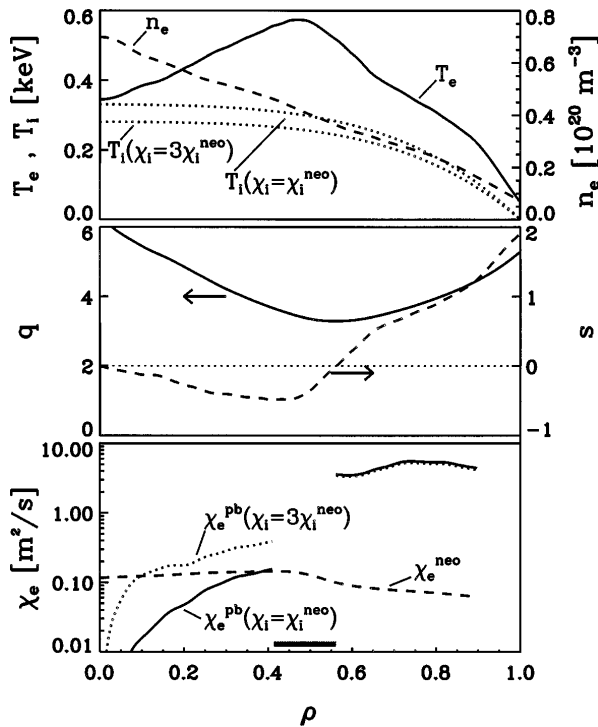


FIG. 6. LPB results for discharge r19950620.021. The upper box shows the smoothed TS profiles of  $T_e(\rho)$  (full line) and  $n_e(\rho)$  (dashed line), and the  $T_i$  profiles used in the analysis (see text, dotted lines). In the middle box  $q$  (full line) and shear parameter  $s$  (dotted line) are plotted, assuming full current penetration. The lower box shows  $\chi_e^{pb}$ , under the two different assumptions for  $T_i$  (full and dotted line), plus the neoclassical prediction of  $\chi_e$  (dashed line). The shaded area indicates the ECH power deposition zone.

The global electron energy confinement time ( $\tau_{E,e}$ ), which in Ohmically heated medium to high density discharges in RTP is 3–5 ms, degrades to 1–3 ms with off-axis heating, and decreases with increasing  $\rho_{dep}$ .

The effective electron thermal diffusivity  $\chi_e^{pb} = q_e / n_e \nabla T_e$  (where  $q_e$  is the electron heat flux) was determined from a local electron power balance (LPB) analysis. Since no  $T_i$  measurements were available, two assumptions for the effective ion heat diffusivity ( $\chi_i^{pb}$ ) were considered: (i)  $\chi_i^{pb} = \chi_i^{neo}$  and (ii)  $\chi_i^{pb} = 3\chi_i^{neo}$ , respectively ( $\chi_i^{neo}$  is the neoclassical ion thermal diffusivity). In the first case  $T_i(0) \approx 500$  eV in Ohmically heated plasmas, in agreement with earlier observations. Case (ii) gives an unrealistically low  $T_i(0) \approx 300$  eV in Ohmic plasmas, resulting in an upper estimate of  $\chi_e^{pb}$  in the area with a hollow  $T_e$  profile. Radiation losses are low ( $\approx 10$ – $20$  kW, measured with bolometry) and are neglected in the present analysis. Full current adaptation has been assumed, which is justified since the profiles were taken about 4 current diffusion times after the switch-on of ECH. For the densities considered,  $\sim 95\%$  of the ECH power is absorbed in the first pass. The power deposition zone has a FWHM of 3 cm

(measured directly and calculated with ray tracing). In the analysis, 100% single pass absorption was assumed and all power is assumed to be deposited in a zone of 3 cm, which was omitted from the LPB analysis. Both assumptions give an upper estimate of the inward electron heat flux inside  $\rho_{dep}$ , hence an upper estimate of  $\chi_e^{pb}$ .

Spitzer resistivity has been assumed in the analysis, and the bootstrap current density ( $j_{boot}$ ) has been neglected. The off-axis heated discharges are only marginally in the banana regime in the region from  $\rho \approx \rho_{dep}$  to  $\rho \approx 0.8$ , and in the plateau regime elsewhere. The combined effect of the neoclassical correction to the resistivity and of  $j_{boot}$  is small, and leads to a slight enhancement of the central current density  $j_0$ .

Figure 6 shows  $\chi_e^{pb}$  for a high density discharge with  $\rho_{dep} = 0.48$ . The calculated  $\chi_e^{pb}$  is close to or below neoclassical for  $\rho < \rho_{dep}$ . Outside the power deposition zone  $\chi_e^{pb} \geq 4$  m<sup>2</sup>/s, the usual anomalous value for RTP.

For discharges just above the threshold density, the LPB analysis yields puzzling results. For such discharges, both assumptions on  $T_i$  cause  $p_{e-i}$  to be smaller than the residual ohmic power density ( $p_\Omega$ ) inside  $\rho_{dep}$ , leading to a net outward electron heat flux, i.e., up the  $T_e$  gradient. Only the unrealistic assumption  $T_i = 0$  would yield a net inward electron heat flux (see Fig. 7). This is a strong indication that off-diagonal elements in the transport matrix (the matrix relating the fluxes to the thermodynamical forces) are responsible for a significant

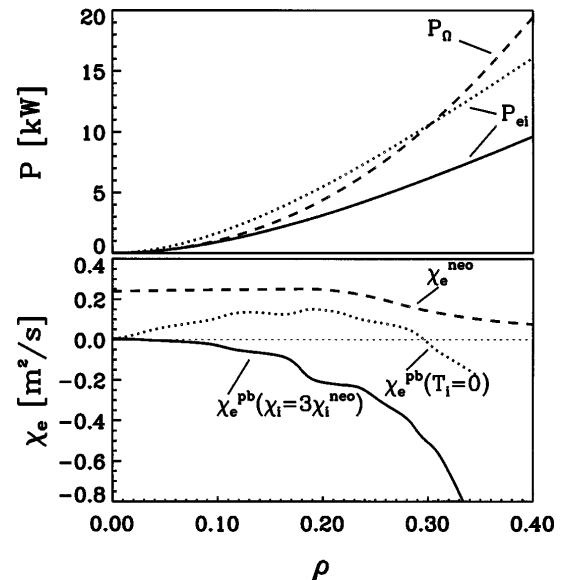


FIG. 7. LPB results for discharge r19950622.053 (see Fig. 3) for the area  $\rho = 0$ – $0.4$ . The upper box shows  $P_\Omega$  (the volume integrated  $p_\Omega$  inside  $\rho$ , dashed line) and  $P_{e-i}$  (volume integrated  $p_{e-i}$  inside  $\rho$ ) under two assumptions for  $T_i$  (see text, full and dotted lines). The lower box shows  $\chi_e^{pb}$  (full and dotted lines), together with the neoclassical prediction of  $\chi_e$  (dashed line). Only the unrealistic assumption  $T_i = 0$  everywhere yields  $P_{e-i}$  larger than  $P_\Omega$ , i.e., a positive  $\chi_e^{pb}$ .

electron heat flux. First results of an experiment in which short heating pulses were applied in the center of the hollow  $T_e$  profile corroborate this view. The time constants associated with the induced rise and decay of  $T_e(0)$  suggest that it is not  $\chi_e$  itself that has changed, but that other, i.e., off-diagonal, terms are crucial in the electron heat flux. At this moment it is an open question which gradient drives this flux. Interestingly, during the evolution of the profiles,  $\nabla T_e$  and the shear  $s$  go from negative to positive and back with different time constants, so that all possible combinations occur. A detailed, time dependent analysis may reveal the role of the current density distribution in driving a heat flux.

About the error on  $\chi_e^{pb}$  inside  $\rho_{dep}$ , the following can be said. The systematic error on  $T_e$  and  $n_e$  from TS is small,  $\leq 10\%$ . Both profiles were smoothed for the LPB analysis, thus removing the effect of statistical errors, and causing the thus calculated  $\chi_e^{pb}$  to be a radially averaged value. Regarding  $q_e$ , the assumptions on  $j$ ,  $T_i$ , and the ECH power deposition were all made such that an upper estimate of  $\chi_e^{pb}$  was obtained.

Transiently hollow  $T_e$  profiles have been observed in other tokamaks, for instance, in JET when off-axis ion cyclotron heating was applied after pellet injection [15]. There, the temporary hollowness was due to thermal inertia and the evolution of the  $T_e \approx T_i$  profile was proven to be diffusive.

There has been earlier evidence of reduced net heat transport in a region of reversed magnetic shear  $s$ , both theoretically [16] and experimentally. An example of the latter is the PEP mode in JET [17] which, however, is again transient in nature. Current drive can in principle sustain steady-state reversed shear, e.g., the lower hybrid enhanced performance regime in Tore Supra [18,19] and JET [20]. Recently, reversed shear configurations have drawn increasing interest as a possible scenario for a fusion reactor [21,22]. In the off-axis ECH discharges in RTP, reduced heat transport does not lead to improved global confinement, because no power is deposited in the region with reduced transport.

In contrast to observations in DIII-D [10], with off-axis ECH in RTP no peaking of  $T_e$  brought about by an inward heat pinch is observed. Still, as in DIII-D, evidence is found for a heat flow against  $\nabla T_e$ . It should be noted, however, that the electron collisionality  $\nu_e^* \approx 1$  in high density discharges in RTP, whereas in DIII-D  $\nu_e^* \ll 1$ ; this difference may provide an explanation for the different observations in DIII-D and RTP: the drift wave transport model that successfully simulated the DIII-D data [23] predicts an inward heat pinch only in the collisionless regime.

In this Letter we have shown that hollow  $T_e$  profiles have been sustained in RTP, thus breaking profile

resilience. Extremely low net electron thermal transport is found in the region with inverted  $T_e$  gradient, in which also the magnetic shear is negative. The steady-state hollow  $T_e$  profile is a unique observation, achieved in RTP because of the very good localization of the 110 GHz 2nd harmonic X mode ECH, and the large ratio  $P_{ECH}/P_\Omega \approx 5$ .

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- [1] B. Coppi, Comments Plasma Phys. Controlled Fusion **5**, 261 (1980).
  - [2] M. Murakami *et al.*, Plasma Phys. Controlled Fusion **28**, 17 (1986).
  - [3] F. Wagner *et al.*, Phys. Rev. Lett. **56**, 2187 (1986).
  - [4] G. Becker, Nucl. Fusion **31**, 663 (1991).
  - [5] F.C. Schüller *et al.*, in *Controlled Fusion and Plasma Physics, Proceedings of the 18th European Conference, Berlin, 1991* (European Physical Society, Geneva, 1991), Vol. 15C, Part IV, p. 185.
  - [6] B.B. Kadomtsev, Sov. J. Plasma Phys. **13**, 443 (1987).
  - [7] Yu.N. Dnestrovskij *et al.*, Nucl. Fusion **31**, 1877 (1991).
  - [8] B.B. Kadomtsev, Plasma Phys. Controlled Fusion **34**, 1931 (1992).
  - [9] J.B. Taylor, Phys. Fluids B **5**, 4378 (1993).
  - [10] C.C. Petty and T.C. Luce, Nucl. Fusion **34**, 121 (1994).
  - [11] A.A.M. Oomens *et al.*, in Proceedings of the 9th Joint Workshop on Electron Cyclotron Emission and Electron Cyclotron Resonance Heating, Borrego Springs, 1995 (to be published).
  - [12] C.J. Barth *et al.*, Rev. Sci. Instrum. **63**, 4947 (1992).
  - [13] C.C. Chu *et al.*, in *Controlled Fusion and Plasma Physics, Proceedings of the 21st European Conference, Montpellier, 1994* (European Physical Society, Geneva, 1994), Vol. 18B, Part III, p. 1248.
  - [14] J.H. Rommers *et al.*, in *Controlled Fusion and Plasma Physics, Proceedings of the 22th European Conference, Bournemouth, 1995* (European Physical Society, Geneva, 1995), Vol. 19C, Part III, p. 421.
  - [15] B. Balet *et al.*, Nucl. Fusion **34**, 1175 (1994).
  - [16] F. Romanelli and F. Zonca, Phys. Fluids B **5**, 4081 (1993).
  - [17] M. Hugon *et al.*, Nucl. Fusion **32**, 33 (1992).
  - [18] G.T. Hoang *et al.*, Nucl. Fusion **34**, 75 (1994).
  - [19] L. Guiziou *et al.*, in *Controlled Fusion and Plasma Physics, Proceedings of the 22th European Conference, Bournemouth, 1995* (European Physical Society, Geneva, 1995), Vol. 19C, Part I, p. 37.
  - [20] F.X. Söldner *et al.*, in Ref. [19], Part IV, p. 113.
  - [21] M.E. Mauel *et al.*, in Ref. [19], Part IV, p. 137.
  - [22] C. Kessel *et al.*, Phys. Rev. Lett. **72**, 1212 (1994).
  - [23] J. Weiland and H. Nordman, Phys. Fluids B **5**, 1669 (1993).