Intrinsic Toroidal Rotation Driven by Turbulent and Neoclassical Processes in Tokamak Plasmas from Global Gyrokinetic Simulations

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Gyrokinetic tokamak plasmas can exhibit intrinsic toroidal rotation driven by the residual stress. While most studies have attributed the residual stress to the parallel-momentum flux from the turbulent $E \times B$ motion, the parallel-momentum flux from the drift-orbit motion (denoted Π_{\parallel}^D) and the $E \times B$ -momentum flux from the $E \times B$ motion (denoted $\Pi_{E \times B}$) are often neglected. Here, we use the global total-*f* gyrokinetic code XGC to study the residual stress in the core and the edge of a DIII-D H-mode plasma. Numerical results show that both Π_{\parallel}^D and $\Pi_{E \times B}$ make up a significant portion of the residual stress. In particular, Π_{\parallel}^D in the core is higher than the collisional neoclassical level in the presence of turbulence, while in the edge it represents an outflux of countercurrent momentum even without turbulence. Using a recently developed "orbit-flux" formulation, we show that the higher-than-neoclassical-level Π_{\parallel}^D in the core is driven by turbulence, while the outflux of countercurrent momentum from the edge is mainly due to collisional ion orbit loss. These results suggest that Π_{\parallel}^D and $\Pi_{E \times B}$ can be important for the study of intrinsic toroidal rotation.

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Tokamak plasmas can rotate toroidally without external momentum input, which is important for future reactors where internal fusion heating is not expected to generate net momentum. Such intrinsic toroidal rotation is driven by the residual stress, which is a momentum flux independent from the toroidal-rotation velocity and its gradient. The gyrokinetic approach is often used to find the residual stress in turbulent plasmas, but its determination can be difficult because turbulence will transport equal amounts of co- and countercurrent momentum, so the net momentum flux is zero unless there is an asymmetry in the parallel direction. Therefore, studies of the residual stress have been active for many years [1-18].

In an electrostatic gyrokinetic plasma, radial transport comes from the drift-orbit motion \boldsymbol{v}_D and the turbulent $\boldsymbol{E} \times \boldsymbol{B}$ motion \boldsymbol{v}_{E} . While most studies have attributed the residual stress to the parallel-momentum flux from \boldsymbol{v}_{E} (the "fluid stress"), the parallel-momentum flux from \boldsymbol{v}_D (the "kinetic stress") and the $E \times B$ -momentum flux from \boldsymbol{v}_E (the "toroidal Reynolds stress") are often neglected in the core. In particular, the kinetic stress is usually assumed to be at a small collisional neoclassical level. However, as will be discussed in this Letter, part of the kinetic stress can be driven by turbulence, which is already observed by several global gyrokinetic simulations [19–23] and studied from a qualitative theory [23]. Numerically, the residual stress is often studied in the local geometry, where the volume-integrated kinetic stress and Reynolds stress vanish due to radial periodicity, but this radial boundary condition no longer exists in the global geometry. The kinetic stress has also been emphasized for the edge rotation [24–27], which is affected by not only turbulence, but also complicated factors such as the realistic geometry with a magnetic X point [24–31], interactions with neutrals [32,33], and ion orbit loss [34–43]. With the advancing computing power, global gyrokinetic simulations with realistic geometry could provide new physics insights for this topic.

In this Letter, we use the global total-f particle-in-cell gyrokinetic code XGC [44] to study the residual stress in a DIII-D H-mode plasma. Both the core and the edge are studied through whole-volume plasma simulation from the magnetic axis to the wall. We initiate the plasma with zero rotation velocity and study the self-generated momentum fluxes. The gyrocenter toroidal angular momentum (TAM) density consists of the parallel-flow part \mathcal{L}_{\parallel} and the $E \times B$ -flow part $\mathcal{L}_{E \times B}$. Their corresponding radial TAM fluxes are denoted by Π_{\parallel} and $\Pi_{E \times B}$, respectively. Numerical results show that both Π^D_{\parallel} (the component of Π_{\parallel} from \boldsymbol{v}_D) and $\Pi_{E \times B}$ make up a significant portion of the residual stress. Using a recently developed "orbit-flux" formulation [45–48], we quantitatively show how Π^D_{\parallel} is driven not only by collisions, but also by turbulence in the core, as well as by collisional ion orbit loss in the edge. Similar results are also found in the core of a larger machine ITER, as discussed toward the end. These results suggest that Π^D_{\parallel} and $\Pi_{E \times B}$ can be important for the study of intrinsic toroidal rotation.

Simulation setup.—We simulate deuterium gyrokinetic ions and drift-kinetic electrons. Their equilibrium density



FIG. 1. (a),(b) The equilibrium density (in units m⁻³) and temperature (in units keV) as a function of normalized poloidal flux ψ_n , and $n_{e0} = n_{i0}$ due to quasineutrality. (c),(d) The TAM density [in units kg/(m · s)] and flux (in units N · m) at t = 0.4 ms from the neoclassical XGCa simulation.

and temperature are adapted from DIII-D shot number 141451 [25–27] and are functions of the poloidal magnetic flux ψ (Fig. 1). The gyrocenter coordinates are cylindrical position $\mathbf{R} = (\mathbf{R}, \varphi, z)$, magnetic moment μ , and parallel momentum p_{\parallel} . The Hamiltonian for species *s* is $H = p_{\parallel}^2/2m_s + \mu B + Z_s e \hat{J}_0 \Phi$, where $\hat{J}_0 \Phi$ is the gyroaveraged electrostatic potential. Define $v_{\parallel} = p_{\parallel}/m_s$, $\hat{\mathbf{b}} = \mathbf{B}/B$, $\mathbf{B}^* = \mathbf{B} + (m_s v_{\parallel}/Z_s e) \nabla \times \hat{\mathbf{b}}$, and $B_{\parallel}^* = \hat{\mathbf{b}} \cdot \mathbf{B}^*$, then the gyrocenter trajectories are given by

$$B_{\parallel}^* \dot{\boldsymbol{R}} = v_{\parallel} \boldsymbol{B}^* + (Z_s e)^{-1} \hat{\boldsymbol{b}} \times \nabla H, \qquad B_{\parallel}^* \dot{p}_{\parallel} = -\boldsymbol{B}^* \cdot \nabla H.$$

Separating *H* into an axisymmetric part $\bar{H} = p_{\parallel}^2/2m_s + \mu B + Z_s e \hat{J}_0 \bar{\Phi}$ and a nonaxisymmetric part $\tilde{H} = Z_s e \hat{J}_0 \bar{\Phi}$, we have $\dot{R} = v_{\parallel} \hat{b} + v_D + v_E$, where $v_{\parallel} \hat{b} + v_D$ is the parallel and the drift-orbit motion and v_E is the $E \times B$ motion from $\bar{\Phi}$. Note that v_D includes not only the grad-*B* and the curvature drift, but also the $E \times B$ drift from $\bar{\Phi}$.

The "total-f" numerical scheme is used for the wholevolume plasma simulation [49], where "f" refers to the gyrocenter distribution F_s , which evolves according to

$$d_t F_s = \partial_t F_s + \dot{\mathbf{R}} \cdot \nabla F_s + \dot{p}_{\parallel} \partial_{p_{\parallel}} F_s = C_s + S_s + N_s, \quad (1)$$

and is allowed to significantly deviate from the equilibrium Maxwellian distribution. Here, C_s describes collisions [50,51], S_s describes heating, and N_s describes neutral ionization and charge exchange [52]. In our simulations, a 1 MW heating is applied to ions in the core to sustain turbulence, and neutral dynamics are included in the edge and the scrape-off layer.

Neither S_s nor N_s generate net momentum in our simulations. Then, the gyrokinetic Eq. (1) has a local

gyrocenter TAM conservation relation [15–19]

$$\partial_t (\mathcal{L}_{\parallel} + \mathcal{L}_{E \times B}) = -\partial_V (\Pi_{\parallel} + \Pi_{E \times B}), \qquad (2)$$

where $V(\psi)$ is the volume inside the flux surface ψ . The TAM densities are calculated as $\mathcal{L}_{\parallel} = \sum_{s} \langle \int d^{3}v F_{s} p_{s\varphi} \rangle$ and $\mathcal{L}_{E\times B} = -(dV/d\psi)^{-1} \int dt \sum_{s} Z_{s} e\Gamma_{s}$, where $\langle ... \rangle$ is the flux-surface average, $p_{s\varphi} = -m_{s}v_{\parallel}\hat{\boldsymbol{b}} \cdot R^{2}\nabla\varphi$ is the TAM from parallel motion, and $\Gamma_{s} = \langle \int d^{3}v F_{s} \dot{\boldsymbol{R}} \cdot \nabla V \rangle$ is the radial gyrocenter flux. The sign of $p_{s\varphi}$ is chosen so that a positive (negative) TAM density corresponds to a co-(counter) current toroidal rotation. The TAM fluxes are calculated as $\Pi_{\parallel} = \sum_{s} \langle \int d^{3}v F_{s} p_{s\varphi}(\dot{\boldsymbol{R}} \cdot \nabla V) \rangle$ and $\Pi_{E\times B} = -\int dV \sum_{s} \langle \int d^{3}v F_{s} \partial_{\varphi} H \rangle$. Since radial transport comes from both \boldsymbol{v}_{D} and \boldsymbol{v}_{E} , we write

$$\Gamma_s = \Gamma_s^D + \Gamma_s^E, \qquad \Pi_{\parallel} = \Pi_{\parallel}^D + \Pi_{\parallel}^E \tag{3}$$

to emphasize their separate contributions.

Simulation results.—First, we use the axisymmetric version of XGC (XGCa) to simulate a neoclassical plasma, where $\Pi^E_{\parallel} = \Pi_{E \times B} = 0$ but Π^D_{\parallel} can be nonzero due to collisions. Starting from a local Maxwellian F_s , the plasma relaxes to a quasisteady state at t = 0.4 ms, when the TAM densities and flux are shown in Figs. 1(c) and 1(d). In the core, $\mathcal{L}_{E \times B} < 0$ due to a negative neoclassical radial electric field E_r , while $\mathcal{L}_{\parallel} > 0$ from the parallel return flow that balances the poloidal $E \times B$ and diamagnetic flow. The neoclassical-level Π^D_{\parallel} is very small in the core, so the TAM density is conserved at each flux surface, $\mathcal{L}_{\parallel} \approx -\mathcal{L}_{E \times B}$. In the edge, $\mathcal{L}_{E \times B}$ has a countercurrent peak at $\psi_n \approx 0.99$ due to the H-mode edge E_r well. Correspondingly, \mathcal{L}_{\parallel} has a cocurrent peak, but the relation $\mathcal{L}_{\parallel} \approx -\mathcal{L}_{E \times B}$ is no longer satisfied due to a dipolar Π^D_{\parallel} in the edge. Throughout the simulation, the edge $\mathcal{L}_{\|}$ shifts in the countercurrent direction at the pedestal top ($\psi_n < 0.98$) and in the cocurrent direction toward the last closed flux surface ($\psi_n = 1$) according to Eq. (2).

Next, we use the 3D version of XGC (XGC1) to simulate a turbulent plasma and the results are shown in Fig. 2. Turbulence is active in the core but decays in the edge due to the *H*-mode E_r well. In the core, turbulence-driven $\mathcal{L}_{E\times B}$ and \mathcal{L}_{\parallel} have similar radially wavelike structures. Note that here \mathcal{L}_{\parallel} and $\mathcal{L}_{E\times B}$ have the same sign, which is different from the XGCa solution $\mathcal{L}_{\parallel} \approx -\mathcal{L}_{E\times B}$. In the edge, turbulent intensity is weak so that the TAM flux is dominated by Π^D_{\parallel} , and the corresponding edge rotation is also similar to the XGCa solution. The observed edge $\Pi^D_{\parallel} \approx -0.3 \text{ N} \cdot \text{m}$ is comparable to that inferred from experiments [26,27], and our simulation results in the edge are qualitatively



FIG. 2. XGC1 simulation results showing (a) amplitude of the turbulent fluctuations, (b) $\mathcal{L}_{E\times B}$, and (c) \mathcal{L}_{\parallel} as a function of *t* (in units ms) and ψ_n . The XGCa solutions of \mathcal{L}_{\parallel} and $\mathcal{L}_{E\times B}$ are subtracted to remove their large peaks in the edge.

consistent with the results using a previous version of XGC with a different setup [25].

The above results showed that both neoclassical and turbulent processes can generate residual TAM fluxes and toroidal rotation in our simulations. In the following, we study the physics behind these momentum fluxes.

Core momentum fluxes.—Figure 3(a) shows E_r , which varies radially and drives differential poloidal rotation known as zonal flows. Since $\mathcal{L}_{E\times B}$ is proportional to E_r , the observed correlation between $\mathcal{L}_{E\times B}$ and \mathcal{L}_{\parallel} can be understood as the correlation between zonal flows and toroidal rotation, which was also seen in other global gyrokinetic simulations [53–56]. As shown in Figs. 3(b) and 3(c), turbulent eddies are tilted according to the local zonal-flow shear, and the corresponding Π_{\parallel}^E and $\Pi_{E\times B}$ oscillate radially. Meanwhile, Π_{\parallel}^D is larger than the neoclassical solution in Fig. 1(d) and tends to be out of phase with Π_{\parallel}^E , in agreement with other global gyrokineticsimulation results [19–23]. Therefore, all the three TAM fluxes should be considered in order to correctly predict the toroidal-rotation evolution in the core [Fig. 3(d)].

We found these TAM fluxes significant in the sense that $|v_i \Pi/aQ_i|$ can be as large as 0.5, meaning they can drive toroidal rotation up to a non-negligible fraction of the ion thermal velocity v_i [12,13]. (Here, *a* is the minor radius and Q_i is the ion heat flux). It is well known that the zonal-flow shear can produce finite correlation between poloidal and parallel wave spectra and hence a nonzero Π_{\parallel}^E [57,58].



FIG. 3. (a) E_r (in units V/m) at t = 1 ms along the outboard midplane. The black dashed lines are flux surfaces where $\partial_r E_r \approx 0$. (b) $\tilde{\Phi}$ (in units V) near the outboard midplane. (c) The corresponding TAM fluxes. (d) Comparison of the TAM conservation relation (2) with numerical results.

However, studies often assumed that $\Pi_{E \times B}$ is smaller than Π_{\parallel}^{E} by a factor $k_r \rho_i B_{\theta} / B$ and Π_{\parallel}^{D} is at a small collisional neoclassical level [1]. Our results showed that these assumptions are not always valid, and we focus on the origin of $\Pi_{E \times B}$ and Π_{\parallel}^{D} in the following.

Using the relation $\partial_{\varphi} = Rb_{\varphi}\hat{\boldsymbol{b}} \cdot \nabla + B^{-1}\hat{\boldsymbol{b}} \times \nabla \psi \cdot \nabla$ and assuming $k_{\parallel} \ll k_{\perp}$ for turbulence, one can show that

$$\partial_V \Pi_{E \times B} \approx (dV/d\psi)^{-1} J^E,$$
 (4)

where $J^E = \sum_s Z_s e \Gamma_s^E$ is the turbulent radial current. This approximation is numerically verified in Fig. 4(a), and can be interpreted as the proportionality between toroidal and poloidal projection of the Reynolds stress. Note that the drift-orbit current $J^D = \sum_s Z_s e \Gamma_s^D$ balances J^E so that the total gyrocenter current is small. By comparing Figs. 3(a) and 4(a), these radial currents also oscillate with the zonal flow. Since a positive gyrocenter current drives E_r in the negative direction and vice versa, the zonal flow is driven by J^E and damped by J^D . As E_r forms according to J^E , toroidal rotation driven by $\Pi_{E\times B}$ will have the same radial profile as E_r according to Eq. (4). This is a novel explanation for the correlation between toroidal rotation and zonal flows.

Although \boldsymbol{v}_D contains the $\boldsymbol{E} \times \boldsymbol{B}$ drift from $\bar{\Phi}$, the corresponding TAM flux is small in our simulations, so most of Π^D_{\parallel} is from the magnetic (grad-*B* and curvature) drift of ions, $\boldsymbol{v}_{\text{mag}} \approx (\mu B + m_i v_{\parallel}^2) \hat{\boldsymbol{b}} \times \nabla \ln B/Z_i \boldsymbol{e}$. Write $\mu B + m_i v_{\parallel}^2 = 2T_{i0} + (\mu B - T_{i0}) + (m_i v_{\parallel}^2 - T_{i0})$ as the contributions from isothermal processes and deviation to



FIG. 4. (a) The radial currents and $\partial_{\psi}\Pi_{E\times B}$ (in units A) at t = 1 ms. The black dashed lines are the flux surfaces plotted in Fig. 3. (b) The isothermal and nonisothermal parts (5) of Π_{\parallel}^{D} in Fig. 3. (c) Orbit-flux calculations (9) for momentum fluxes across the $\psi_{n} = 0.76$ core flux surface. (d) Comparison with the direct calculation of Π_{\parallel}^{D} from XGC1 using Eq. (6).

Maxwellian distribution in the perpendicular and parallel directions, the kinetic stress $\Pi^D_{\parallel} \approx \langle \int d^3 v F_i p_{i\varphi} \boldsymbol{v}_{\text{mag}} \cdot \nabla V \rangle$ can then be written as

$$\Pi^{D}_{\parallel} = \Pi_{\text{mag,iso}} + \Pi_{\text{mag},\perp} + \Pi_{\text{mag},\parallel}, \qquad (5)$$

with the three terms from $2T_{i0}$, $\mu B - T_{i0}$, and $m_i v_{\parallel}^2 - T_{i0}$, respectively. It is straightforward to show that $\Pi_{\text{mag,iso}} \propto -T_{i0} \langle \gamma_{\parallel} \sin \theta \rangle$, where $\gamma_{\parallel} = \int d^3 v F_i v_{\parallel}$ is the ion parallel flux density and θ is the poloidal angle. Therefore, a nonzero $\Pi_{\text{mag,iso}}$ arises due to the up-down asymmetry in γ_{\parallel} . As discussed in Ref. [23], such asymmetric γ_{\parallel} can be driven by the divergence of the turbulent radial flux, i.e., $\Pi_{\text{mag,iso}} \propto \partial_r \Gamma_i^E$, which explains the radially oscillatory behavior of Π_{\parallel}^D in Fig. 3. However, as shown in Fig. 4(b), both $\Pi_{\text{mag,}\perp}$ and $\Pi_{\text{mag,}\parallel}$ are comparable to $\Pi_{\text{mag,iso}}$, so the qualitative theory from Ref. [23] alone cannot explain the turbulence-driven Π_{\parallel}^D in our simulations. Further, we found the contributions to $\Pi_{\text{mag,}\perp}$ and $\Pi_{\text{mag,}\parallel}$ from temperature fluctuations to be small, so they must come from higherorder moments in the ion distribution.

Turbulent origin of Π_{\parallel}^D in the core.—Although the observed Π_{\parallel}^D cannot be simply explained from the low-order fluid moments of F_i , we can still numerically illustrate the turbulent origin of Π_{\parallel}^D using a recently developed "orbit-flux" formulation [45,46]. By definition, the kinetic stress at flux surface ψ is

$$\Pi^{D}_{\parallel} = \frac{2\pi}{m_{i}^{2}} \oint \sqrt{g} d\theta \, d\varphi \int dp_{\parallel} d\mu B^{*}_{\parallel} F_{i} p_{i\varphi} \boldsymbol{v}_{D} \cdot \nabla \psi, \quad (6)$$

where $\sqrt{g} = |\nabla \psi \times \nabla \theta \cdot \nabla \varphi|^{-1}$. Since drift-orbit motion $v_{\parallel} \hat{\boldsymbol{b}} + \boldsymbol{v}_D$ conserves the canonical TAM $\mathcal{P}_{\varphi} = p_{i\varphi} - Z_i e \psi$ and the energy \bar{H} , we can use $(\mu, \mathcal{P}_{\varphi}, \bar{H})$ to label all drift orbits that cross the flux surface ψ . Changing variables from (p_{\parallel}, θ) to $(\mathcal{P}_{\varphi}, \bar{H})$, we obtain

$$\Pi^{D}_{\parallel} = \frac{2\pi}{Z_{i}em_{i}^{2}} \int d\mu d\mathcal{P}_{\varphi} d\bar{H} \oint d\varphi \left(F_{i}^{\text{out}} - F_{i}^{\text{in}}\right) p_{i\varphi}.$$
 (7)

Here, it is assumed that each drift orbit crosses the flux surface twice, one at the incoming point and the other at the outgoing point, and we define F_i^{in} and F_i^{out} to be the ion distribution at these two points, respectively. For each drift orbit, $\Delta F_i = F_i^{\text{out}} - F_i^{\text{in}}$ can be calculated as an orbit integration from the incoming point to the outgoing point at fixed time t,

$$\Delta F_{i} = \int d\tau \left(C_{i} + S_{i} + N_{i} - \tilde{\mathbf{R}} \cdot \nabla F_{i} - \tilde{p}_{\parallel} \partial_{p_{\parallel}} F_{i} - \partial_{t} F_{i} \right), \quad (8)$$

where $\hat{\boldsymbol{R}} = \boldsymbol{v}_E$, $\tilde{p}_{\parallel} = -\boldsymbol{B}^* \cdot \nabla \tilde{H} / B_{\parallel}^*$, and the integration is along drift orbits parameterized by τ . Combining (7) and (8), we write Π_{\parallel}^D as the summation of "orbit fluxes,"

$$\Pi^{D}_{\parallel} = \Pi_{\rm col} + \Pi_{\rm src} + \Pi_{\rm neut} + \Pi_{\rm turb} + \Pi_{t}.$$
 (9)

A similar procedure can be applied to Γ_i^D to obtain

$$\Gamma_i^D = \Gamma_{\rm col} + \Gamma_{\rm src} + \Gamma_{\rm neut} + \Gamma_{\rm turb} + \Gamma_t.$$
(10)

Equations (9) and (10) are called "orbit-flux" formulations, which show that Π_{\parallel}^{D} and Γ_{i}^{D} are not only driven by collisions (which is the focus of the conventional neoclassical theory), but also by heating, neutral dynamics, turbulence, and time evolution of the plasma along collisionless drift orbits. Our simulated plasma is in the low-collisionality banana regime, with mostly collisionless drift orbits up to the last closed flux surface.

We numerically implemented this formulation in XGC [47,48]. As an example, we look at Π_{\parallel}^{D} at the $\psi_{n} = 0.76$ flux surface and the results are in Figs. 4(c) and 4(d). The dominant contribution to Π_{\parallel}^{D} are the turbulent term Π_{turb} and the associated time evolution of the plasma Π_{i} , while effects from collisions and heating are small. Also, the orbit-flux calculation (9) agrees well with the direct calculation of Π_{\parallel}^{D} in XGC1 using (6), which demonstrated that it is implemented with good numerical accuracy. These results quantitatively confirmed that the higher-than-neoclassical-level Π_{\parallel}^{D} in the XGC1 simulation is indeed driven by turbulence.

Edge momentum fluxes.—In the edge region of our simulation, turbulence is weak and Π^D_{\parallel} is mainly driven



FIG. 5. (a),(c) The particle fluxes (in units s⁻¹) and momentum fluxes across the $\psi_n = 0.992$ edge flux surface. Shown are the separate contribution from confined orbits and loss orbits. (b),(d) Orbit-flux calculations (9) and (11) for the loss orbits.

by neoclassical processes. Note that the edge is subject to ion orbit loss, where some drift orbits do not form closed loops but connect the confined region to the divertor leg or the vessel wall. Considering the separate contributions from loss orbits and the remaining confined orbits (which form closed loops), we write orbit fluxes as, e.g.,

$$\Gamma_{\rm col} = \Gamma_{\rm col}^{\rm loss} + \Gamma_{\rm col}^{\rm conf}, \qquad \Pi_{\rm col} = \Pi_{\rm col}^{\rm loss} + \Pi_{\rm col}^{\rm conf}.$$
(11)

Results for an edge flux surface $\psi_n = 0.992$ are shown in Fig. 5. For the particle flux, we find $\Gamma^{\text{loss}} > 0$ and $\Gamma^{\text{conf}} < 0$. In other words, while gyrocenter ions leave the plasma following the loss orbits, they also enter the plasma following the confined orbits. For the momentum flux, however, both Π^{loss} and Π^{conf} are countercurrent and they add up to $\Pi^D_{\parallel} \approx -0.3 \text{ N} \cdot \text{m}$ in the edge. These results are consistent with each other, namely, most loss orbits are countercurrent and the remaining confined orbits are overall cocurrent, so that both $\Gamma^{\text{loss}} > 0$ and $\Gamma^{\text{conf}} < 0$ result in countercurrent momentum fluxes. The loss-orbit fluxes are mainly caused by collisional scattering of ions into the loss orbits, while effects from turbulence and neutrals are small. Therefore, the outgoing countercurrent momentum fluxes in the edge are mainly from collisional ion orbit loss within our simulation. Finally, note that the self-consistent orbitloss driven Π^D_{\parallel} determines $\partial_t \mathcal{L}_{\parallel}$ in the edge, which is different from simple orbit-loss models that determine \mathcal{L}_{\parallel} itself [40,41].

Conclusions.—In summary, global total-*f* gyrokinetic simulations showed that Π^D_{\parallel} is higher than the collisional neoclassical level in the presence of turbulence, and both Π^D_{\parallel} and $\Pi_{E\times B}$ make up a significant portion of the residual stress in a DIII-D H-mode plasma. Using the orbit-flux formulation, we identified the mechanisms that drive Π^D_{\parallel} ,

including turbulence in the core and collisional ion orbit loss in the edge. It is often assumed that \boldsymbol{v}_D gives rise only to neoclassical transport, which is driven solely by collisions and is smaller than the turbulent transport from \boldsymbol{v}_E . Our results showed that this assumption is not always valid, because part of the radial transport from \boldsymbol{v}_D can be driven by turbulence. In Supplemental Material [59], we provide an ordering estimate for Π^D_{\parallel} and argue that it can be comparable to Π^E_{\parallel} ; we also report similar results in simulations of electrostatic turbulence in a larger machine ITER. These results suggest that Π^D_{\parallel} and $\Pi_{E\times B}$ can be important for the study of intrinsic toroidal rotation, and global gyrokinetic simulations could lead to further new physics insights for this topic.

The data that supports the findings of this study are openly available at Princeton Data Commons Discovery [62].

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