

$^{12}\text{C}(n, 2n)^{11}\text{C}$ cross section from threshold to 26.5 MeVM. Yuly,^{*} T. Eckert, and G. Hartshaw*Department of Physics, Houghton College, Houghton, New York 14744, USA*

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The $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross section was measured from just below threshold to 26.5 MeV using the Pelletron accelerator at Ohio University. Monoenergetic neutrons, produced via the $^3\text{H}(d, n)^4\text{He}$ reaction, were allowed to strike targets of polyethylene and graphite. Activation of both targets was measured by counting positron annihilations resulting from the β^+ decay of ^{11}C . Annihilation gamma rays were detected, both in coincidence and singly, using back-to-back NaI detectors. The incident neutron flux was determined indirectly via $^1\text{H}(n, p)$ protons elastically scattered from the polyethylene target. Previous measurements fall into upper and lower bands; the results of the present measurement are consistent with the upper band.

DOI: [10.1103/PhysRevC.97.024613](https://doi.org/10.1103/PhysRevC.97.024613)**I. INTRODUCTION**

The $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction may be a useful and robust neutron diagnostic for measuring the areal density ρR of a deuterium-tritium (DT) implosion, which is an important parameter in determining the implosion compression of an inertial confinement fusion (ICF) burn. Not only is the reaction sensitive to ρR , but because the reaction is only sensitive to neutrons above 20 MeV, it is immune to primary neutrons (14.1 MeV) and down-scattered neutrons. To use this method, ultra-pure graphite disks placed within the ICF reaction chamber become activated by tertiary neutrons via the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction. The 511-keV gamma rays emitted by the ^{11}C disk during positron annihilation are subsequently counted in an area far away from the target chamber and used to obtain the tertiary neutron yield. The diagnostic is well suited for the harsh EMP environment produced during an ICF implosion and high gamma and x-ray background [1,2]. Furthermore, the ^{11}C half-life is sufficiently long compared to the graphite extraction time which is on the order of a few minutes. This allows for the counting process to begin soon after the ICF shot prior to radioactive cooling of the graphite. It is important to note that the tertiary yield cannot be determined without a good knowledge of the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction cross sections in this

energy range. Hence the importance of these new cross-section measurements.

A measurement of the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction in this energy range is also important for calculations of the rate of cosmogenic ^{11}C production, because the uncertainty in this cross section represents the largest source of systematic error in these calculations. In turn, the presence of cosmogenic ^{11}C in deep underground mines limits the detectability of *pep* and CNO solar neutrinos in several neutrino experiments [3].

Figure 1 shows the previous measurements in the energy range between threshold (at 20.2957 ± 0.0010 MeV [4]) and 35 MeV, as well as predicted cross sections from Dimbylow [5], which are from a nuclear optical model calculation using fits to experimentally measured total elastic and inelastic cross sections. The cross sections tend to follow two separate bands which differ by as much as a factor of two across the neutron energy range of interest. The upper band comprises measurements from Anders *et al.* [6] and Welch *et al.* [7] and calculations of Dimbylow [5], while the lower band is the measurements from Brolley *et al.* [8], Brill *et al.* [9], Soewarsono *et al.* [10], and Uno *et al.* [11].

It is difficult to see what might be causing the results for these experiments to fall into two distinct bands. Table I summarizes the published essential features of each previous experiment. There does not seem to be any obvious division between the bands on the basis of technique, type of neutron

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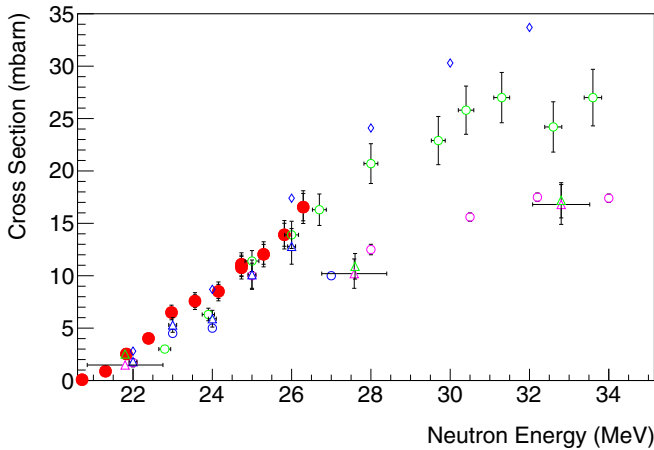


FIG. 1. Cross sections for the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction near threshold. The empty symbols are previously published data: Brolley *et al.* (blue circles) [8], Brill *et al.* (pink circles) [9], Anders *et al.* (green circles) [6], Welch *et al.* (blue triangles) [7], Soewarsono *et al.* (pink triangles) [10], Uno *et al.* (green triangles) [11], and optical model calculations of Dimbylow (blue diamonds) [5]. The solid red symbols are from this experiment using the polyethylene target and both NaI detectors in coincidence; the larger of the associated error bars indicates the increase in overall uncertainty when the uncertainty in incident neutron energy is included.

source, type of target, method of neutron flux determination, or type of activation measurement.

Because it is not clear why the previous measurements disagree, the present experiment was designed to reduce or eliminate possible sources of systematic uncertainty that may have affected previous results. In the present experiment, monoenergetic neutrons produced using the $^3\text{H}(d, n)^4\text{He}$ re-

action were allowed to strike carbon-containing targets of polyethylene and graphite. When these neutrons induced the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction, ^{11}C nuclei were produced, which later decayed via β^+ emission with a half-life of 20.364 ± 0.014 min [12]. After an activation period, the targets were removed to counting stations, where both the singles and coincidence rates of 511-keV gamma rays resulting from positron annihilation were used to determine the number of ^{11}C nuclei present. To determine the neutron flux, protons from neutron-proton elastic scattering were simultaneously counted in a ΔE -E detector telescope.

The present experiment has a number of advantages.

- (1) The electrostatic accelerator provides an extremely stable and nearly monoenergetic deuteron beam, which, when used with a very thin titanium tritide target gives intrinsic neutron energy spread of less than about 0.2 MeV in the 20- to 30-MeV neutron range.
- (2) By using a recoil proton telescope with ΔE -E silicon detectors, the present experiment has the ability to identify and select only $^1\text{H}(n, p)$ elastic recoil protons of the correct energy. Because the solid angle is well defined and the intrinsic efficiency is nearly 100% for the silicon detectors, an absolute determination of the neutron flux is possible using well-measured np elastic scattering cross sections.
- (3) Two targets, graphite and the polyethylene proton production target, irradiated and counted simultaneously, allow a consistency check. Moreover, because the polyethylene activation target is also the proton production target for the telescope, problems that would result from the neutron flux being measured in a different place than the activation target are eliminated.

TABLE I. Previous measurements of the $^{12}\text{C}(n, 2n)$ cross section in the energy range 20–30 MeV.

Year	Experiment	Accelerator	Neutron source	Target	Neutron flux	Activation measurement
1952	Brolley <i>et al.</i> [8]	10.5 MeV deuterons from cyclotron	$^3\text{H}(d, n)$ gas cell, neutron energy selected by angle	Poly-ethylene foils	Calculated from $^3\text{H}(d, n)$ cross section	Geiger counter calibrated with RaD+E
1961	Brill <i>et al.</i> [9]	20 MeV deuterons from cyclotron, Pt foil degrader	$^3\text{H}(d, n)$ (Zr foil) and $^2\text{H}(d, n)$ (gas)	Carbon	TOF energy–angle distribution	Geiger counter calibrated with ^{197}Au
1981	Anders <i>et al.</i> [6]	7–16 MeV deuterons from cyclotron, Be foil degrader	$^3\text{H}(d, n)$ (Ti foil)	Reactor graphite	Stilbene crystal recoil proton spect.	Annihilation $\gamma\gamma$ coincidence using NaI detectors
1981	Welch <i>et al.</i> [7]	Tandem Van de Graaff	$^3\text{H}(d, n)$	Natural carbon	No information available	Ge(Li) Detector calibrated with ^{22}Na
1992 1996	Soewarsono <i>et al.</i> [10] Uno <i>et al.</i> [11]	20–40 MeV protons from cyclotron	$^7\text{Li}(p, n)$ quasi-monoenergetic	^7Li on graphite	From activation of Li target	HPGe detector, Li in-out subtraction

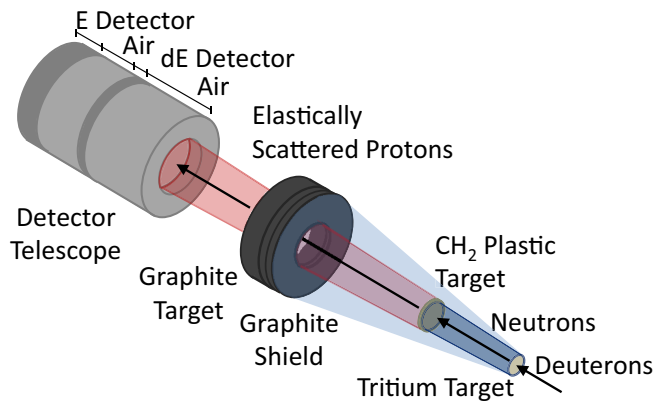


FIG. 2. The experimental setup for activating targets. Deuterons traveling down the beam pipe were collimated, then struck the tritium target to produce neutrons. These neutrons activated the graphite and polyethylene targets via the $^{12}\text{C}(n, 2n)$ reaction. The polyethylene target also acted as the converter for the recoil proton telescope. The recoiling protons were identified and counted using a dE-E telescope.

- (4) Counting the activated targets by placing them between the circular faces of two cylindrical matched NaI crystals gives maximum solid angle and therefore maximum absolute efficiency for counting the relatively small number of ^{11}C decays, thereby reducing counting uncertainty and the effect of unwanted background. Requiring a coincidence rather than using a single detector eliminates most background 511-keV gamma ray events that do not come from ^{11}C decay.
- (5) A careful study was made of the absolute full-peak efficiency of the counting system. A Monte Carlo code was developed to calculate this efficiency for both singles and coincidence mode geometries, the results of which were validated by comparisons to a number of ancillary experiments. This allowed a consistency check to be made by simultaneously measuring the cross sections using both the coincidence and single count rates.

II. DESCRIPTION OF THE EXPERIMENT

The cross sections were measured for energies between about 19.7 and 26.4 MeV using the 4.5 MV Tandem Pelletron electrostatic accelerator at Ohio University. As shown in Fig. 2, deuterons were accelerated to energies between 3.1 and 9.1 MeV and allowed to strike a 472.86 GBq tritium target that was perpendicular to the beam and located at the end of the beam pipe just upstream of an aluminum end window. Analyzing magnet image slits restricted the spread in deuteron beam energy to about 5 keV. Deuteron beam currents were typically between 0.8 and 1.0 μA . The tritium was deposited as titanium tritide on a 49-mm diameter 1-mm thick OFHC copper substrate at a density of about 2000 $\mu\text{g}/\text{cm}^2$ over a circular active area of 30-mm diameter. The end window where the target was mounted was cooled by a stream of air. The target assembly was attached to the beamline with a bellows so that the target could be rotated in a circular path in a plane perpendicular to the beam direction. The radius of rotation was approximately 1 cm and the period of rotation was about 0.3 s

which spread the beam heating and target sputtering over a larger area on the target without compromising the geometry of the experiment. Before striking the target, the deuteron beam was defocused by a pair of quadrupole magnets located 275 and 315 cm upstream, and allowed to pass through a 1.27-cm diameter collimator 45-cm upstream of the target. This was to ensure that the beam spot on the target was relatively large, uniform, and in a known and reproducible location. These characteristics minimized local heating of the target and were also needed for simulations described below.

A 50-cm long steel optical bench with modified positioners was used to hold the detectors and targets in fixed positions at an angle of 0° with respect to the beamline. The targets were aligned to the optical center of the beamline using the procedure discussed below.

Neutrons leaving the tritium could strike a 1.64-mm thick, 2.54-cm diameter high-density polyethylene target with its upstream face located 7.0 cm from the tritium target and a 7.62-cm diameter 0.89-cm thick disk of high purity graphite with a 17.46-mm hole drilled in through its center with its upstream face 14.4 cm from the tritium target. In contact with the upstream face of the graphite target, a pair of graphite disks stopped any protons scattered from the polyethylene from reaching the graphite target. The total thickness of this graphite shield was 3.85 mm, with the other dimensions being the same as the graphite target disk. The mass densities of the polyethylene and graphite targets were measured to be 0.957 ± 0.008 and 1.842 ± 0.012 g/cm^3 , respectively.

A proton telescope consisting of a 300- μm thick, 150-mm² ion implanted silicon dE detector and a 5000- μm thick, 200-mm² drifted-lithium silicon E detector was placed behind the hole in the graphite target, so that protons coming from the polyethylene could be viewed. The entrance of the dE detector was covered by a 0.005-mm-thick aluminum foil to keep out ambient light. The entire detector assembly was housed in an aluminum tube with wall thickness of about 2.9 mm and diameter of 3.47 cm. Preamplifiers and spectroscopy amplifiers located near the detectors in the experimental hall sent pulses to a FastComTech MPA-3 multiparameter system which digitized and recorded the pulse heights and timing. The system also recorded the deuteron beam current.

Figure 3 shows two-dimensional (2D) histograms of the pulse height in the dE detector versus the E detector. The $^1\text{H}(n, p)$ protons elastically scattered from the polyethylene can be easily identified by their energy loss in the two detectors. At 26.4 MeV the background count rate was about 3% of the rate with the polyethylene target in place, which was approximately 3.5 protons/s with a beam current of about 1 μA . Over the course of the experiment, radiation damage caused the width of the proton peak in the E detector to gradually increase along with the leakage current.

A special circuit provided a separate count of the number of coincidence events, and gated the individual detector pulses that were input to analog-to-digital converters (ADCs) that were part of the FAST ComTec MPA-3 multiparameter system. The live time, which was typically about 98%, was confirmed by comparing the number of coincidence events recorded by the computer with the number counted by a separate hardware circuit.

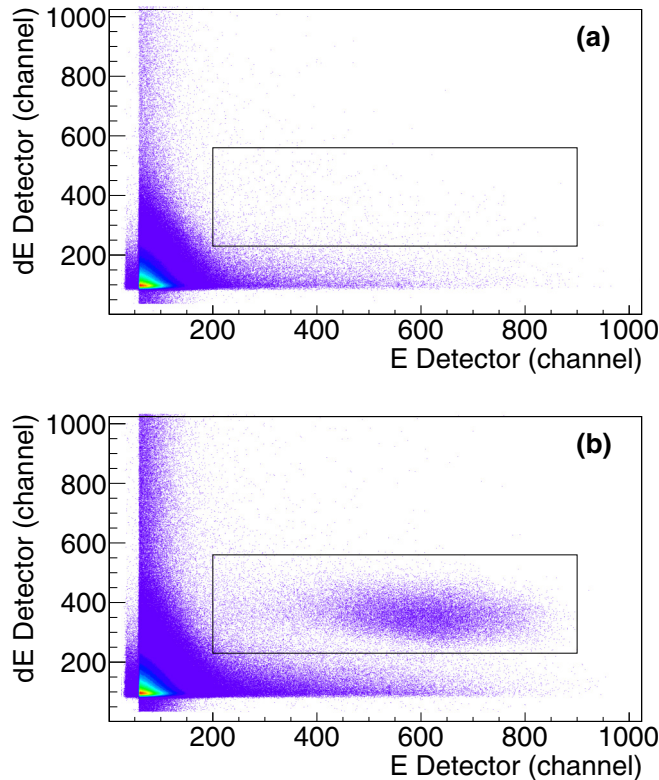


FIG. 3. (b) Histogram of dE versus E for 23.7-MeV neutrons striking the polyethylene target. The proton island is clearly visible. The marked region of interest (black) indicates the elastically scattered protons. (a) Same, but with the polyethylene target removed.

A 12.7-cm diameter, 5.08-cm thick NE-213 liquid scintillator neutron monitor was located 300 cm from the tritium target at an angle of a 71.4° to beam left. To identify neutron pulses from gamma ray signals in the monitor, a pulse shape analysis was made using a Mesytec MPD4 pulse shape discriminator module. These signals were also recorded by the MPA-3 multiparameter system.

Activated targets were counted at three counting stations located in a room far from the accelerator target area to reduce background counts. After each was activated simultaneously, the graphite target disks, shields, and polyethylene targets were placed between pairs of 7.62-cm diameter by 7.62-cm thick NaI detectors. Pairs of detectors and the graphite targets were held with their axes aligned inside almost equal diameter acrylic tubes. The polyethylene targets, which were much smaller in diameter, were affixed with adhesive tape to the center of one of the detectors, which was marked. Pulses from all of these detectors were digitized by a FAST ComTec MPA-4 system, which recorded the pulse heights and timing information. Coincidence events consisting of two back-to-back 0.511-keV gamma rays from positron annihilation were selected (as shown in Fig. 4) and counted as a function of time. This allowed the growth curve of ^{11}B to be measured and fit to determine the number of ^{11}C nuclei present. The gamma rays from the graphite shields were also counted in a separate station consisting of two high-purity germanium detectors to look for activation from contaminants in the graphite.

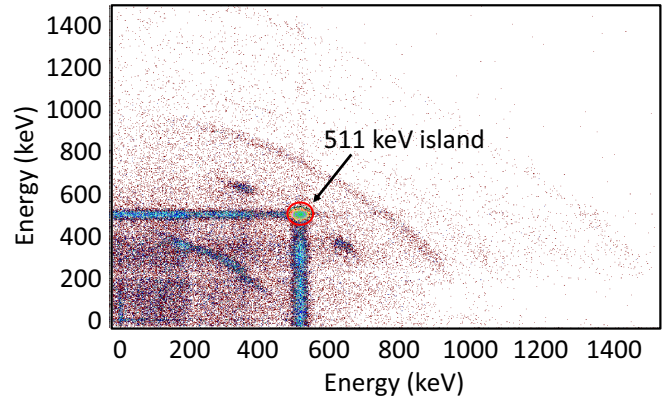


FIG. 4. A 2D histogram showing the pulse height in each NaI detector for coincidence events. The events in the large coincidence peak were counted for 50-s intervals to produce the ^{11}B growth curve.

III. PROCEDURE

The silicon detector telescope and targets were positioned at 0° to the beamline using a theodolite that had been previously aligned with the collimator in the beam pipe and monuments in the target room.

To ensure the polyethylene target holder was centered and the neutron distribution was axially symmetric, a duplicate target was cut in half and the proton count rate was measured to approximately 2% statistical uncertainty with the half target in each of four positions: top, bottom, left, and right. The target position was changed by simply rotating it in the holder, and the ratio of protons to integrated charge on the tritium target, which should be constant for perfect axial symmetry, was found to vary by less than 10%. This small deviation from a uniform neutron distribution on the polythene target would only result in a small correction to the already small extended target correction described below.

For each energy setting, prior to activating the targets, a study was made of the beam defocussing and positioning of the beam along the axis of the beam pipe. This step is important because the distribution of the beam on the target must be input to the simulation of the experiment described below. Experimentally, it was observed that steering the beam off of the central axis of the collimator would change the number of protons per integrated charge on the tritium target and the ratio of protons to neutrons detected in the neutron monitor. To place the beam on the central axis of the quadrupoles, the steering of the beam was adjusted such that the ratio of detected protons to neutrons was minimally sensitive to the quadrupole current (less than a 5% change for a quadrupole current change of 25%). The deuteron beam was also monitored with a beam profiler upstream from the target. These procedures ensured that the beam was very closely aligned with the optical beam axis and centered on the target.

Following these quadrupole tests, a shield, graphite target, and a polyethylene target were placed in the target holders to be activated for about 1.5 h, during which time the proton telescope pulses were recorded. Three identical sets of targets were available to be used consecutively to allow enough time for any longer lived activated contaminants to

decay between uses, although no contaminants were detected. When the deuteron beam was stopped, after sufficient time to allow the room radiation dose rate to fall to an acceptable level, the targets were hand-carried to the counting room and placed in the counting stations. The time between when irradiation stopped and when target counting commenced was typically 4–5 min. Each sample was then counted for approximately 2 h, binned into 50-s intervals. The dead time for each time bin was recorded and used to correct the growth curve.

The background proton count rate was measured at each energy setting by removing the polyethylene target and counting for approximately 30 min. A separate graphite disk target and shield was used exclusively for this purpose.

IV. ANALYSIS

A. Overview

The ^{11}C decays in the activated polyethylene and graphite targets were counted using pairs of NaI detectors “sandwiching” each target, capable of counting the 511-keV gamma rays from positron annihilation in both singles and coincidence modes simultaneously. This allowed the ^{11}B growth curves from $^{11}\text{C} \rightarrow ^{11}\text{B} + e^+ + \nu$ to be measured for both single and coincidence events, and fit with the exponential growth function,

$$R(t_c) = R_0(1 - e^{-\lambda t_c}) + At_c + B, \quad (1)$$

where $R(t_c)$ is the sum of all the positron annihilation events counted up to time t_c , R_0 is the total number of detectable ^{11}C decays, $\lambda = 20.364 \pm 0.014$ min [12] is the decay constant for ^{11}C , and $At_c + B$ is the integral of the constant rate of background events. The number of counts N_0 that would be obtained if counting began immediately at the end of activation is

$$N_0 = R_0 e^{\lambda t_{\text{trans}}}, \quad (2)$$

where t_{trans} is the time between the end of activation and the start of counting, in other words, the time required to transfer the samples to the counting station. The total number of ^{11}C nuclei formed in the target is

$$N_{^{11}\text{C}} = \frac{N_0}{\epsilon}, \quad (3)$$

where ϵ is either the absolute full-peak coincidence or singles efficiency, depending on how the growth curve was generated.

Cross sections σ for $^{12}\text{C}(n, 2n)$ were extracted using the above quantities as well as the background-subtracted rate of elastically scattered protons detected N_p , and the activation time t since

$$\sigma = \frac{N_0}{\epsilon} \frac{1}{T_C} \frac{\lambda}{1 - e^{-\lambda t}} \left(\frac{N_p}{N_n} \right) \frac{1}{N_p}, \quad (4)$$

where T_C is the target thickness in terms of carbon nuclei (carbon nuclei/cm²). The quantity (N_p/N_n) is the ratio of elastically scattered protons detected N_p , to the number of neutrons striking the polyethylene or graphite target N_n . This ratio was calculated numerically for the experiment geometry for each target using the known $^3\text{H}(d, n)^4\text{He}$ [13] and $^1\text{H}(n, p)n$ elastic scattering [14,15] cross sections.

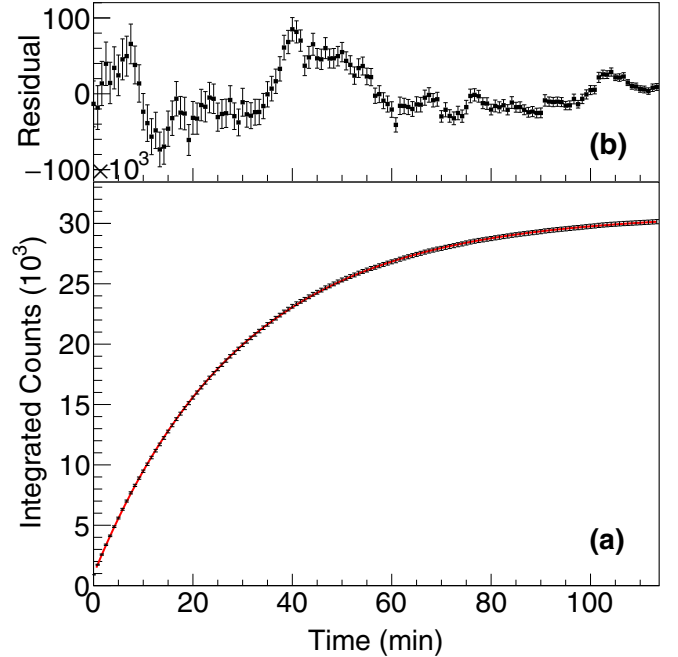


FIG. 5. (a) Fit of Eq. (1) to the coincidence growth curve and (b) residuals for the graphite target activated by 26.3-MeV neutrons. The uncertainties shown were calculated using the integrated number of counts (a) and the incremental number for each time bin $\sqrt{N_i - N_{i-1}}$ (b).

B. Growth curves

To determine R_0 , Eq. (1) was fit to the growth curves for the polyethylene and graphite targets using the C++ ROOT [16] TMINUIT class implementation of the MINUIT package [17]. Fits were made using single events from each NaI detector individually and also for coincidence events, resulting in six semi-independent measurements of the cross section. For the polyethylene target, which is much thinner than the graphite, at the highest incident neutron energy setting about 7000 ^{11}C decays were counted over the 2-h period.

Figure 5 shows a typical fit of Eq. (1) to a coincidence growth curve. These growth curves were created by integrating the number of events in the 511-keV peak up to time t_c , and plotting the integral as a function of t_c . The exponential nature of the growth curve was clear for coincidence events, but because of the large number of background events relative to the number of ^{11}C decays, the single growth curves for the thin polyethylene targets were nearly a straight line. Nevertheless, the value of R_0 could still be extracted from these fits and used to determine the cross section using Eq. (4), albeit with a larger uncertainty.

C. Determination of (N_p/N_n)

To figure out the cross section for the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction using Eq. (4), the incident neutron flux N_n striking each target was determined from the number of protons N_p using

$$N_n = \frac{1}{\left(\frac{N_p}{N_n}\right)} N_p, \quad (5)$$

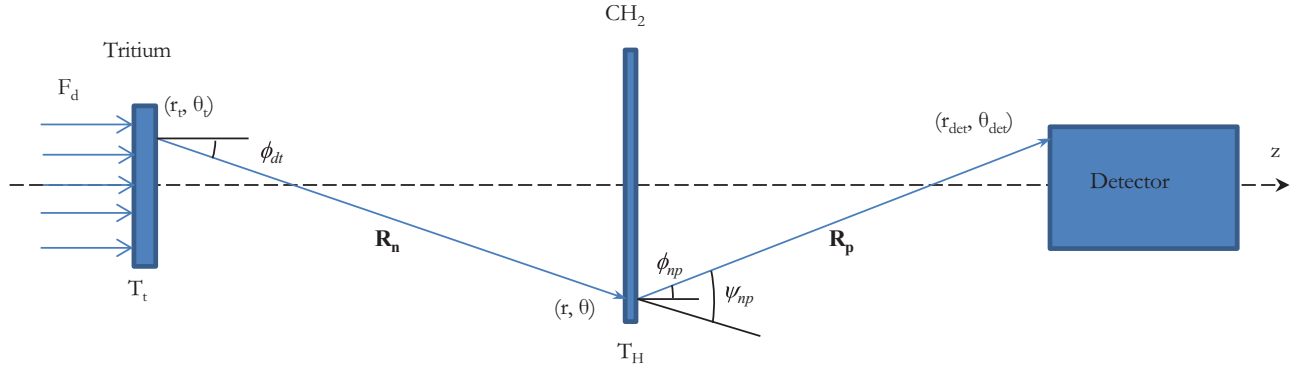


FIG. 6. Schematic diagram showing quantities used to calculate the (N_p/N_n) proton to neutron ratio for extended targets. A neutron can travel from an infinitesimal area element at polar coordinates (r_t, θ_t) on the tritium target to an area element at polar coordinates (r, θ) on the polyethylene target along vector \mathbf{R}_n . A proton can then travel from there to the area element at polar coordinates $(r_{\text{det}}, \theta_{\text{det}})$ on the detector along vector \mathbf{R}_p . The total number of neutrons or protons were determined by integrating over the surfaces of the involved targets or detector. The z coordinate axis is along the center of the beamline.

where the quantity in parenthesis, (N_p/N_n) , the ratio of the number of protons detected to neutrons striking the target was calculated purely from the experiment geometry.

To calculate the ratio (N_p/N_n) for each target, several simplifying assumptions were made to which corrections were later applied. Assume that the tritium target is a point source of monoenergetic neutrons, isotropic in the laboratory frame, with flux N (neutrons/s/sr). In that case, the number of neutrons/s (N_n) striking the polyethylene ($N_n = N_{\text{CH}_2}$) and graphite ($N_n = N_G$) targets would be given by $N_{\text{CH}_2} = N\Omega_{\text{CH}_2}$ and $N_G = N\Omega_G$ where Ω_{CH_2} and Ω_G are the solid angles of the plastic and graphite targets, respectively, if the effect of the finite thickness of the targets on the solid angle is neglected. The number of protons/s (N_p) detected by the proton telescope can be obtained if the polyethylene target is then treated as a point source for the purpose of scattering protons into the proton telescope. Assuming that the cross section for ${}^1\text{H}(n, p)$ elastic scattering, $\sigma_{np}(\psi_{np}, E_n)$, which depends on the scattering angle ψ_{np} , and the incident neutron energy E_n , is roughly constant over the angles subtended by the targets, and is equal to the cross section at 0° at the nominal neutron energy, yields

$$N_p = \sigma_{np}(0^\circ) T_H N_n \Omega_p, \quad (6)$$

where T_H is the thickness (hydrogen nuclei/area) of the polyethylene target, and Ω_p is the solid angle of the proton telescope. In this simple approximation, therefore, the calculated ratio of the rates for protons detected by the proton telescope to neutrons hitting the polyethylene target is

$$\left(\frac{N_p}{N_n}\right)_{\text{CH}_2} = \frac{N_p}{N_{\text{CH}_2}} = \sigma_{np}(0^\circ) T_H \Omega_p, \quad (7)$$

and for the graphite target,

$$\left(\frac{N_p}{N_n}\right)_G = \frac{N_p}{N_G} = \sigma_{np}(0^\circ) T_H \frac{\Omega_{\text{CH}_2} \Omega_p}{\Omega_G}. \quad (8)$$

Clearly, the approximation that the tritium and polyethylene targets can be treated as point sources is incorrect, and a more correct solution must include the fact that the neutrons leaving the tritium target can have a range of angles and still reach the

polyethylene and graphite targets, and that the energies and cross sections for these neutrons depend on the neutron angle. Moreover, the protons coming from the polyethylene, which is actually an extended source, can also have a range of angles and still strike the proton telescope, and the cross sections and energies of the protons reaching the telescope depend on the proton angle.

A number of corrections were applied to this simple calculation of the (N_p/N_n) proton to neutron ratio. The first correction accounts for the fact that the targets were not point sources but actually extended targets. The ${}^1\text{H}(n, p)$ elastic scattering and DT fusion cross sections both depend on the scattering angle, as do the energies of the scattered neutrons and protons. To account for these effects, the surface of each target and the silicon detector were divided into infinitesimally small area elements, each of which was then treated as a point source. The total number of protons or neutrons hitting a target or detector was determined by integrating over the surface area of each target, as depicted in Fig. 6.

The rate of neutrons hitting the polyethylene target is

$$N_{\text{CH}_2} = \int_0^{2\pi} \int_0^R \int_0^{2\pi} \int_0^{R_t} \sigma_{dt}(\phi_{dt}, E_d) \times F_d T_t r_t dr_t d\theta_t \frac{\cos \phi_{dt}}{R_n^2} r dr d\theta, \quad (9)$$

where R_t is the radius of the beam spot on the tritium target (0.635 cm), R is the radius of the polyethylene target, the incident deuteron energy is E_d , the deuteron flux (deuterons/area/time) is F_d , the thickness of the tritium target (${}^3\text{H}$ nuclei per unit area) is T_t , and the surface area of the tritium target is A_t . The cross section for DT fusion at the neutron angle ϕ_{dt} for incident deuteron energy E_d is $\sigma_{dt}(\phi_{dt}, E_d)$ where $\phi_{dt} = \cos^{-1}(\mathbf{R}_n \cdot \hat{\mathbf{z}}/R_n)$.

The number of neutrons impacting the graphite target (N_G) was calculated using the same formula by integrating over the surface area of the graphite target rather than the polyethylene target, remembering to include the central hole.

The rate of protons striking the proton telescope from ${}^1\text{H}(n, p)$ elastic scattering of DT neutrons from protons in the

plastic target is given by

$$\begin{aligned}
 N_p = & \int_0^{2\pi} \int_0^{R_{\text{det}}} \int_0^{2\pi} \int_0^R \int_0^{2\pi} \int_0^{R_t} \sigma_{np}(\psi_{np}, E_n(\phi_{dt}, E_d)) \\
 & \times \sigma_{dt}(\phi_{dt}, E_d) F_d T_t \frac{T_H}{\cos \phi_{dt}} r_t dr_t d\theta_t \frac{\cos \phi_{dt}}{R_n^2} r dr d\theta \\
 & \times \frac{\cos \phi_{np}}{R_p^2} r_{\text{det}} dr_{\text{det}} d\theta_{\text{det}}, \quad (10)
 \end{aligned}$$

where R_{det} is the radius of the proton detector, $E_n(\phi_{dt}, E_d)$ is the DT neutron energy, which depends on the deuteron energy and DT neutron angle ϕ_{dt} , and ψ_{np} is the np scattering angle, given by $\psi_{np} = \cos^{-1}(\mathbf{R}_n \cdot \mathbf{R}_p / R_n R_p)$.

The hole through the center of the graphite target was slightly too small for all of the protons to pass through unobstructed, thereby causing a collimating effect. To correct this, if in the integral any proton path intersected the graphite disk it was excluded from the integral. This changed N_p by less than 0.5%.

A C++ program was written to evaluate these integrals numerically using the rectangle method. Each integral was divided into 150 steps, the number of steps chosen based on the rate of convergence to give around 1% uncertainty in the proton-to-neutron ratio. The scattering angles for the DT fusion and $^1\text{H}(n, p)$ elastic scattering reactions for each neutron and proton path were determined for each step. No neutron scattered at an angle of greater than around 17° was able to hit the graphite target. No proton scattered at an angle greater than around 25° could hit the detector. Because these extreme angles were very unlikely, most paths had cross sections fairly close to the 0° approximation.

The cross-section measurement was insensitive to the Ti:T ratio and the overall tritium activity of the tritium target, because the neutron flux was measured directly using the proton telescope. However, the neutrons striking the graphite and polyethylene targets are slightly reduced in energy and have a broadened energy distribution as a result of the deuterons losing energy prior to interacting with the tritium, and the resulting neutrons leaving at angles greater than 0° . The thickness of the tritium target was also divided into 150 steps, and for each step in thickness and angle the energy of the neutrons striking the targets was calculated. The calculated neutron spectra were used to correct the nominal neutron energies. To do this, for neutron energies above about 21.5 MeV, where the $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross sections are large enough to allow, a quadratic polynomial was fit to the measured $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross sections as a function of uncorrected nominal neutron energy. This was used with the calculated neutron energy spectrum to predict the expected ^{11}C distribution in each target. Then the total ^{11}C in the target and the preliminary cross-section fit were used to determine the corrected neutron energy—that is, if all the neutrons had this energy, they would give the same number of ^{11}C nuclei as the actual neutron distribution. This process resulted in a maximum downward shift in neutron energy of less than 1.3% (0.28 MeV) which occurred for 5.57 MeV deuterons. The FWHM of the energy neutron energy distribution striking the targets was typically about 0.3 MeV. For deuteron energies below 21.5 MeV, where

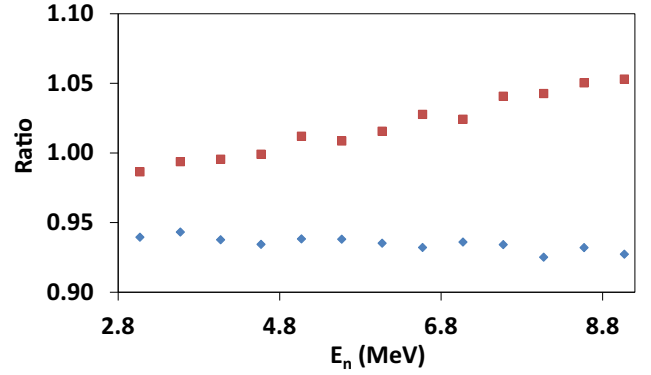


FIG. 7. The ratio of (N_p/N_n) determined by integration using Eqs. (10) and (9) to the value determined using the simple approximate approach of Eqs. (7) and (8) for the graphite (red squares) and the polyethylene targets (blue diamonds).

the $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross section is essentially zero, the average was used to estimate the corrected neutron energy.

The proton energy variation with angle for protons entering the proton telescope is wider, but still only 5 MeV for 26-MeV protons, which allowed the protons to be easily identified in the E- Δ E telescope.

Figure 7 shows how the corrected ratio (N_p/N_n) , computed using Eqs. (10) and (9), compares to the approximate value determined using Eqs. (7) and (8). The net effect of all these corrections was always less than about 7%.

D. Absolute full-peak efficiencies for singles and coincidence

Because the activated targets were sandwiched between the NaI detectors to maximize the count rate, the efficiency was very sensitive to geometry. A Monte Carlo simulation was created using the GEANT4 toolkit [18,19] to model both the graphite and polyethylene target geometries, including other decay modes, surrounding materials, and Compton scattering. To validate this code, it was used to predict efficiencies for other geometries which were then tested experimentally. These experiments will be described in more detail in a later paper.

In the first set of tests, an associated particle technique was used to allow the absolute singles and coincidence full-peak efficiencies to be determined using an uncalibrated ^{22}Na positron source. In this technique the positrons were stopped in a plastic scintillator, signaling that a pair of 511-keV gamma rays were released. The fraction of plastic scintillator events for which the NaI detector also detected a 511-keV gamma ray in coincidence is the absolute efficiency. A third detector was used to correct for summing with the 1.274-MeV gamma rays from the ^{22}Na . The geometry of the plastic scintillator and ^{22}Na source were simulated, and compared with predicted singles and coincidence absolute full-peak efficiencies as a function of source-to-detector distance and radial source position on the face of the detector. The code predictions agreed with the measurements to within approximately 4.7% RMS percent difference for coincidence, and 8.6% for singles.

The second set of tests used an approximately 3.7-kBq NIST calibrated ^{68}Ge source, with activity measured to 1.7%. For

TABLE II. The absolute full-peak efficiencies used in the cross-section calculation.

Target	Configuration	Efficiency
Graphite	Coincidence	0.0494 ± 0.0027
	Detector 1	0.0981 ± 0.0054
	Detector 2	0.1049 ± 0.0058
Polyethylene	Coincidence	0.1568 ± 0.0078
	Detector 1	0.1655 ± 0.0091
	Detector 2	0.1656 ± 0.0091

the initial test, the source was sandwiched between two copper disks to stop the positrons, nearly simulating a point source. Again the geometry was simulated and predicted singles and coincidence absolute full-peak efficiencies were compared with measurements as a function of source-to-detector distance and radial source position on the face of the detector, agreeing to about 5.3% RMS percent difference for coincidence, and 6.6% for singles.

Finally, the ^{68}Ge source was placed between graphite disks of the same diameter and approximate thickness as in the experiment, and the full-peak efficiencies were measured as a function of radial source position. In this experiment, which is the closest to the actual measurement geometry, the predicted efficiencies agreed with the measurement to 6.5% RMS percent difference for coincidence, and 1.4% for singles.

The GEANT simulation code was used to study the effect of misaligning at axes of the targets and NaI detectors. In the worst case, which was for coincidence measurements using the graphite target, an offset of 1 mm of the target resulted a 3.1% change in the full-peak efficiency.

Based on these results, a systematic uncertainty of 5.5% was assigned for the predicted absolute full-peak efficiencies for the targets used in this experiment, which are listed in Table II.

Given the large size of the graphite target used in this experiment, the Monte Carlo simulation code MCNP5 [20] was used to reproduce the distribution of ^{11}C nuclei in the graphite disk. In this simulation, monoenergetic neutrons were emitted uniformly and isotropically from a circular region on the flat tritium target, with a radius equal to that of the last upstream collimator on the deuteron beam. The interactions of these neutrons with the graphite disk were simulated using the standard 6000.60c cross-section library, and resulting number of ^{11}C nuclei were mesh tallied as a function of position within the graphite. The simulation showed, relatively independently of the neutron energy, that the number of ^{11}C was reduced to about 75% at the downstream face relative to the upstream face of the graphite, and smoothly fell by approximately 5% from the center to the edge. Simulating this ^{11}C distribution in the GEANT4 code slightly increased the singles absolute full-peak efficiency for the detector with more ^{11}C near it and reduced the other side; for coincidences the efficiency was increased by nearly 6%.

E. Measurement uncertainty

The uncertainty in the cross section from Eq. (4) was calculated using the normal propagation of uncertainty rules

TABLE III. Contribution to the cross-section uncertainty from each term in Eq. (11) corresponding to the factors in Eq. (4). For the statistical uncertainties in the number of ^{11}C and incident neutrons, the ranges given are for energies above about 21.5 MeV, where the cross section is large enough for this to be meaningful.

Source	Percent uncertainty
Uncertainty in N_0	
Graphite target	
^{11}C decays counted in coincidence	0.3–0.9%
^{11}C decays counted in NaI1 detector	0.2–1.0%
^{11}C decays counted in NaI2 detector	0.2–1.0%
Polyethylene target	
^{11}C decays counted in coincidence	0.7–2.3%
^{11}C decays counted in NaI1 detector	1.0–9.5%
^{11}C decays counted in NaI2 detector	1.0–5.5%
Uncertainty in NaI detector efficiency ϵ	5.5%
Uncertainty in ^{12}C areal number density T_C	
Graphite target	1.9%
Polyethylene target	3.8%
Uncertainty in incident neutrons N_n	
Graphite target	6.9%
Polyethylene target	4.6%
Uncertainty in $(\lambda/1 - e^{-\lambda t})$	0.6%

to get

$$(\delta\sigma)^2 = \sigma^2 \left\{ \left(\frac{\delta N_0}{N_0} \right)^2 + \left(\frac{\delta\epsilon}{\epsilon} \right)^2 + \left(\frac{\delta T_C}{T_C} \right)^2 + \left(\frac{\delta N_n}{N_n} \right)^2 + \left(\frac{1}{\lambda} - \frac{te^{-\lambda t}}{1 - e^{-\lambda t}} \right)^2 (\delta\lambda)^2 \right\}, \quad (11)$$

where the uncertainty in the activation time t is so small it was neglected. The range of uncertainty values for each term in Eq. (11) is shown in Table III, and the results are included in the uncertainties given in Table IV and Figs. 1 and 8. The estimates for statistical uncertainty in the number of ^{11}C decays result from the error matrix calculated by TMINUIT in fitting Eq. (3). The range of values in Table III result from the large change in the number of ^{11}C produced in each target, because the $^{12}\text{C}(n,2n)^{11}\text{C}$ cross section rises rapidly near threshold. The uncertainty in the number of neutrons, determined from $(N_n/N_p)N_p$, depends on the statistical uncertainty in N_p , the number of protons detected (about 1.1%), the uncertainty in the $^1\text{H}(n,p)n$ elastic scattering cross section σ_{np} [14,21] (about 0.7%), the uncertainty in T_H , the polyethylene ^1H number density (about 3.5%), and the solid angle uncertainties for the polyethylene and graphite targets, and the proton telescope (about 2.8%, 1.4%, and 2.9%, respectively). The effect of misaligning the axes of the polyethylene and graphite targets was simulated and found to result in approximately an additional 4.1% uncertainty in the number of neutrons striking the graphite target. Because the neutrons striking the polyethylene target determines the number of protons, this systematic effect does not affect the polyethylene uncertainty. The uncertainties in the ^{12}C and ^1H areal number densities

TABLE IV. Total $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross sections obtained from the graphite and polyethylene targets, for coincidence and singles counting. Also shown is the weighted mean for each energy. The uncertainties quoted are discussed in the text.

Deuteron energy (MeV)	Graphite				Polyethylene			
	Energy (MeV)	Coincidence (mb)	Detector 1 (mb)	Detector 2 (mb)	Energy (MeV)	Coincidence (mb)	Detector 1 (mb)	Detector 2 (mb)
3.57	20.06	0.00 ± 0.00	−0.05 ± 0.01	0.02 ± 0.01	20.10	0.00 ± 0.01	−0.03 ± 0.13	−0.20 ± 0.01
4.07	20.67	0.08 ± 0.01	0.07 ± 0.01	0.10 ± 0.01	20.71	0.08 ± 0.01	0.50 ± 0.11	0.12 ± 0.01
4.57	21.27	0.78 ± 0.32	0.75 ± 0.31	0.70 ± 0.31	21.31	0.88 ± 0.32	0.79 ± 0.33	1.49 ± 0.32
5.07	21.79	2.69 ± 0.43	4.03 ± 0.51	3.78 ± 0.49	21.83	2.53 ± 0.41	4.04 ± 0.49	3.72 ± 0.41
5.57	22.35	4.37 ± 0.57	4.23 ± 0.56	3.96 ± 0.54	22.39	4.02 ± 0.52	4.14 ± 0.53	3.83 ± 0.52
6.07	22.92	6.76 ± 0.77	6.56 ± 0.76	6.12 ± 0.73	22.97	6.49 ± 0.70	6.00 ± 0.69	6.44 ± 0.70
6.57	23.51	8.19 ± 0.92	7.72 ± 0.88	7.25 ± 0.85	23.56	7.58 ± 0.80	7.43 ± 0.80	7.48 ± 0.80
7.07	24.11	8.89 ± 1.01	8.53 ± 0.98	7.83 ± 0.93	24.15	8.50 ± 0.90	8.18 ± 0.88	8.49 ± 0.90
7.57	24.69	12.34 ± 1.31	11.62 ± 1.26	11.17 ± 1.22	24.74	11.09 ± 1.10	10.70 ± 1.08	10.77 ± 1.10
7.57	24.69	12.19 ± 1.30	9.66 ± 1.11	9.04 ± 1.06	24.74	10.77 ± 1.08	8.58 ± 0.95	9.59 ± 1.08
8.07	25.25	13.45 ± 1.44	12.62 ± 1.37	11.81 ± 1.31	25.29	12.04 ± 1.20	11.44 ± 1.16	12.75 ± 1.20
8.57	25.77	15.91 ± 1.67	13.86 ± 1.51	13.05 ± 1.44	25.82	13.91 ± 1.36	13.36 ± 1.32	12.57 ± 1.36
9.07	26.25	18.16 ± 1.89	16.80 ± 1.78	16.07 ± 1.72	26.29	16.55 ± 1.56	16.43 ± 1.56	16.94 ± 1.56

(T_C and T_H , respectively) for each of the targets was estimated based on careful measurements of the target dimensions and mass.

The uncertainty in the corrected incident neutron energy was assumed because of the uncertainty in the thickness of the tritium target, which was about $\pm 0.7\%$. The neutron energy uncertainty was propagated into the $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross-section uncertainty using the slope of a polynomial fit to the nominal $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross sections as a function of energy. The resulting cross-section uncertainties, which ranged from 0.31 mb to 0.85 mb, were added in quadrature to the other uncertainties described above.

V. DISCUSSION OF EXPERIMENTAL RESULTS

The total $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross sections obtained from the graphite and polyethylene targets, for coincidence and singles counting, are displayed in Table IV. The agreement between cross sections determined using different targets, geometries, efficiencies, and detectors is quite good; the overall RMS percent difference from the mean for energies above 22 MeV, where there are enough statistics to be meaningful, is 6.2%. The best individual measurement is expected to be the coincidence measurement using the polyethylene target, because the background rate is reduced for the coincidence measurement, allowing a more robust fit to the growth curve, and because the same target is used for $^1\text{H}(n, p)$ elastic scattering, eliminating systematic uncertainties in determining the neutron flux.

Figure 8 plots the cross sections measured in this experiment for different combinations of targets and detectors. Figure 8(a) shows the overall agreement between all of the measurements made at each neutron energy. The agreement is good, but with the graphite coincidence measurement being systematically high. Figures 8(b), 8(c) and 8(d) compare just the polyethylene and graphite cross sections for coincidence and singles measurements using each detector. Comparing measurements

with different targets made using the same detector(s) shows agreement within experimental uncertainty, which is a fairly stringent test because the geometry and target material is not the same, supporting the calculated detector efficiency and geometry correction. The measurement at 24.7 MeV was repeated, the first measurement made near the middle of the experiment, and the second at the end. In this case, the coincidence measurements at the middle and end of the experiment agree when comparing graphite to graphite and polyethylene to polyethylene, but the graphite result is higher than the polyethylene for both measurements, slightly outside of the error bars.

It is clear from Fig. 1 that the cross sections from the present experiment fall along the high side of the upper curve previously set by the measurements of Anders *et al.* [6] and Welch *et al.* [7] and calculations of Dimbylow [5]. The predictions of Dimbylow [5] are within our error bars, as are, for the most part, the measurements from Anders *et al.* [6] and Welch *et al.* [7].

VI. CONCLUSION

The cross sections for $^{12}\text{C}(n, 2n)^{11}\text{C}$ have been measured using an activation technique from threshold to 26.3 MeV with an uncertainty of approximately 9%–12% for the higher energies. Previous measurements disagree, tending to fall into upper and lower bands. The results of the present experiment agree with the upper band.

Accurate cross sections may allow the $^{12}\text{C}(n, 2n)$ reaction to be used as neutron diagnostic for ICF. During an ICF implosion, primary and tertiary DT fusion reaction neutrons are produced. Because the ion plasma temperatures in these thermonuclear implosions are typically in the keV range, the primary DT neutron energy is Q -value driven. The energy distribution of primary neutrons is typically peaked around 14.1 MeV and has a small thermodynamic width which is broadened further via straggling by the compressed fuel. These

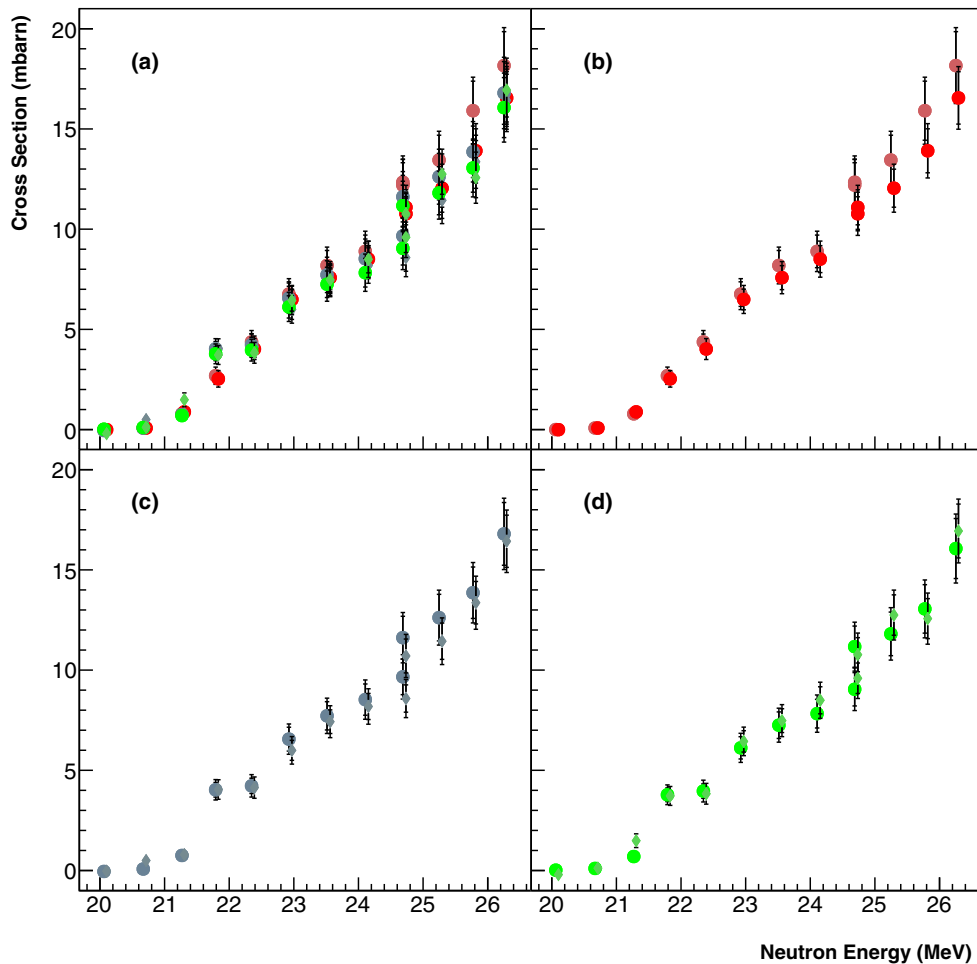


FIG. 8. Comparison of the cross sections measured in this experiment using the graphite (circles) and polyethylene (diamonds) targets for (b) both NaI detectors in coincidence (red), and (c) NaI detector 1 (blue) and (d) NaI detector 2 (green) in singles mode. (a) Shows cross sections determined using both coincidence and singles information for both targets on the same plot. For each cross section, the larger of the associated error bars indicates the increase in overall uncertainty when the uncertainty in incident neutron energy is included.

neutrons are copious and exceed the production of tertiary neutrons by six to seven orders of magnitude. Conversely, neutrons which are produced by DT fusion reactions caused by up-scattered MeV DT fuel generate tertiary neutrons with energies in the 10- to 32-MeV range. Because the $^{12}\text{C}(n, 2n)$ reaction is insensitive to energies below 20 MeV, the primary 14.1-MeV DT and down-scattered primary neutrons cannot react with the carbon. Only tertiary neutrons in the 20- to 32-MeV range react making this method useful as a possible ICF tertiary neutron diagnostic.

To use this method, ultrapure graphite disks placed within the ICF reaction chamber become activated by tertiary neutrons via the $^{12}\text{C}(n, 2n)^{11}\text{C}$ reaction. The ^{11}C in the disk subsequently decays via positron emission, and the 511-keV annihilation gamma rays are then counted in coincidence using a detector system far from the target chamber. The gamma counts can then be used to obtain the tertiary neutron yield, limited by the uncertainty in the $^{12}\text{C}(n, 2n)^{11}\text{C}$ cross section.

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