

Brief Reports

Brief Reports are short papers which report on completed research or are addenda to papers previously published in the *Physical Review*. A Brief Report may be no longer than $3\frac{1}{2}$ printed pages and must be accompanied by an abstract.

Yields of fission products produced by thermal-neutron fission of ^{243}Cm

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(Received 23 April 1986)

On the basis of measured yields for 72 gamma rays and known nuclear data, cumulative fission-product yields were deduced for 69 fission products having half-lives between 36 seconds and 65 days representing 41 mass chains created during thermal-neutron fission of ^{243}Cm .

As part of an ongoing program¹⁻³ we have measured yields for thermal-neutron fission of the rare isotope ^{243}Cm . The sample used contained ~ 75 ng of curium isotopically enriched to $> 99\%$ in the ^{243}Cm isotope by a special isotope separator.⁴ Sample preparation and irradiation by thermal neutrons, gamma-ray detection, data accumulation, and data reduction followed the same procedures as detailed previously.^{2,5} The only substantive differences were (a) use of the Oak Ridge High Flux Isotope Reactor in addition to the Oak Ridge Research Reactor as the neutron source, (b) more recent⁶⁻¹⁵ nuclear data characterizing decay of several of the fission products, and (c) corrections to the experimental yields due to fissions of ^{239}Pu in the sample at the time of the experiment.¹⁶

Five irradiations of different durations were used. The numbers of fissions, n_f , created during each irradiation were deduced with an overall uncertainty of $\pm 7.2\%$ from

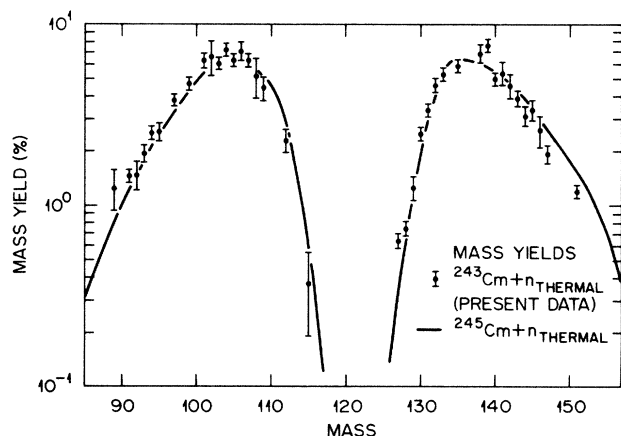


FIG. 1. Mass-yield distribution for $n_{\text{thermal}} + ^{243}\text{Cm}$ derived from the present measurements. Also shown is a smoothed curve representing the $n_{\text{thermal}} + ^{245}\text{Cm}$ mass distribution.

the sample mass, from measured⁴ thermal fission cross section and resonant integral values, and from the known thermal-neutron fluence.

Deduced cumulative fission-product yields (CFY) for the sample, including total absolute uncertainties, are collected in Table I. These CFY were corrected for the contributions from fission of the ^{239}Pu in the sample; the penultimate column of the table gives the ^{239}Pu CFY from the current U.S. ENDF/B evaluation¹⁷ used for these corrections. The ^{239}Pu in the sample accounted for 12% of the fission events. The final column of Table I gives the deduced CFY for thermal-neutron fission of ^{243}Cm .

Deduction of total-mass (i.e., chain) yields (MY) from individual CFY requires knowledge of fractional cumulative yields (FCY). We adopted the FCY vs $(Z - Z_{\text{UCD}})$ relationships deduced² for ^{245}Cm , but, however, computing Z_{UCD} values¹⁸ for ^{243}Cm as the independent variable to determine the FCY for each fission product in Table I. Mass yields were deduced for 38 masses, and they are exhibited in Fig. 1 compared with a curve representing MY for ^{245}Cm obtained from the evaluation.¹⁷ The comparison shown in the figure indicates that the absolute normalization of the ^{243}Cm data appears to be accurate to within $\pm 7.2\%$ uncertainty associated with n_f . One may also observe that the mass distribution for ^{243}Cm is quite similar to that for ^{245}Cm . Indeed, evaluated¹⁷ mass distributions (for thermal-neutron fission) of the pairs ^{233}U - ^{235}U , ^{239}Pu - ^{241}Pu , and ^{249}Cf - ^{251}Cf are also quite similar, much more so than distributions for fissioning systems of different total Z .

We express our appreciation to C. E. Bemis for providing our sample material suitably encapsulated, and to T. R. England (LANL) for providing an up-to-date version of the evaluation of Ref. 17. This research was sponsored by the Office of Basic Energy Sciences, U. S. Department of Energy, under Contract DE-ACO5-84OR21400 with Martin Marietta Energy Systems, Inc.

TABLE I. Deduced cumulative fission-product yields for thermal-neutron fission of ^{243}Cm .

Fission product	E_γ (keV)	Gamma-ray branching ratio (%) ^a	Cumulative fission-product yield		
			Sample	(%) $^{239}\text{Pu}^b$	^{243}Cm
^{89}Rb	1031.9	58.0±5.0	1.29±0.17	1.60±0.27	1.24±0.32
^{91}Sr	1024.3	33.5±0.7	1.57±0.12	2.49±0.05	1.45±0.14
$^{91}\text{Y}^m$	555.6	95.1±0.1	0.91±0.07	1.43±0.05	0.84±0.08
^{92}Sr	1383.9	90.0±9.9	1.63±0.23	2.98±0.12	1.45±0.26
^{93}Sr	590.2	67.4±1.2 ^c	1.98±0.18	3.64±0.22	1.76±0.20
^{94}Sr	1428.1	95.4±0.4	2.04±0.19	3.26±0.20	1.88±0.22
^{94}Y	918.2	49.0±5.0 ^d	2.57±0.35	4.33±0.17	2.34±0.39
^{95}Zr	756.7	54.6±0.5	2.61±0.19	4.88±0.10	2.31±0.22
^{95}Nb	765.8	99.8±0.1	3.05±0.22	4.88±0.10	2.81±0.25
^{97}Zr	743.4 ^e	92.8±0.3 ^e	3.94±0.28	5.26±0.15	3.77±0.32
^{97}Nb	657.9	98.3±0.1	3.91±0.28	5.34±0.11	3.70±0.32
$^{98}\text{Nb}^m$	787.2	93.2±0.2 ^f	0.33±0.03	0.03±0.02	0.37±0.04
$^{99}\text{Tc}^m$	140.5	87.2±0.7 ^g	5.20±0.65	5.43±0.11	5.17±0.74
^{99}Mo	140.5	90.7±0.6	4.81±0.35	6.20±0.13	4.64±0.40
^{101}Mo	192.1	19.2±1.1 ^c	5.81±0.73	5.99±0.18	5.79±0.83
^{101}Tc	306.9	88.0±4.4 ^h	6.47±0.64	6.00±0.17	6.53±0.72
$^{102}\text{Tc}^m$	475.1	85.0±2.0 ^f	0.07±0.04	0.32±0.02 ⁱ	0.04±0.04
^{102}Tc	475.1	6.3±1.0	6.52±1.23	6.05±0.49 ⁱ	6.58±1.39 ^j
^{103}Tc	136.0	16.3±2.9 ^k	5.58±1.14	6.95±0.28	5.40±1.29
^{103}Ru	497.1	90.9±0.7	6.18±0.45	6.95±0.14	6.08±0.51
^{104}Tc	358.0	89.0±5.0	6.90±0.52	6.01±0.36	7.02±0.59
^{105}Tc	143.3	15.7±1.0 ^l	5.54±0.42	5.54±0.61	5.54±0.47
^{105}Ru	724.3	48.0±1.0	6.31±0.53	5.59±0.16	6.41±0.60
^{105}Ru	469.4	1.80±0.07	6.07±0.55	5.59±0.16	6.13±0.62
^{105}Rh	318.9	19.2±0.2	6.41±0.47	5.59±0.11	6.52±0.54
^{106}Tc	270.1	56.0±3.0	4.93±0.52	3.58±0.39	5.10±0.59
^{107}Ru	194.0	14.3±3.4 ^f	5.73±1.52	3.25±0.78	6.06±1.72
^{107}Rh	302.9	65.9±4.6	5.87±1.08	3.26±0.36	6.22±1.22
^{108}Ru	165.0	28.0±7.0	3.93±1.03	2.03±0.33	4.18±1.18
^{109}Rh	326.5	62.0±7.0	4.03±0.58	1.69±0.19	4.34±0.66
^{112}Ag	617.4	42.0±5.0	1.89±0.28	0.12±0.05	2.12±0.31 ^m
$^{115}\text{In}^m$	336.2	45.9±0.1	0.32±0.15	0.06±0.01	0.35±0.17
^{127}Sb	685.7	36.6±0.5	0.62±0.05	0.50±0.04	0.64±0.06
^{128}Sn	482.2	62.3±6.2	0.36±0.07	0.57±0.05	0.34±0.08
$^{128}\text{Sb}^m$	754.0	99.8±0.2	0.58±0.07	0.63±0.04	0.58±0.08
^{128}Sb	754.0	99.8±0.2	0.19±0.02	0.06±0.04	0.21±0.02
^{129}Sb	812.8	45.0±4.5	1.15±0.15	1.35±0.08	1.13±0.17
^{130}Sn	779.8	59.1±3.9 ⁿ	0.34±0.08	0.91±0.07	0.26±0.09
$^{130}\text{Sb}^m$	793.4	86.0±5.0 ^f	1.08±0.13	1.28±0.10	1.05±0.15
$^{130}\text{Sb}^m$	839.5	99.8±0.1	0.93±0.09	1.28±0.10	0.89±0.10
$^{130}\text{Sb}^o$	330.9	77.8±3.9 ^f	0.90±0.10	0.50±0.02	0.95±0.11
^{130}Sb	793.4	99.8±0.1 ^f	0.84±0.08	0.50±0.02	0.89±0.09
^{130}Sb	839.4	99.8±0.1	0.84±0.08	0.50±0.02	0.89±0.09
^{131}Sb	943.6	44.0±4.4 ^p	1.70±0.23	2.53±0.15	1.59±0.26
$^{131}\text{Te}^m$	852.2	21.3±0.9	1.16±0.10	1.04±0.04	1.17±0.11
^{131}Te	149.8	68.9±0.9 ^c	2.28±0.26	2.99±0.08	2.19±0.30
^{131}I	364.5	82.5±0.4	3.41±0.25	3.87±0.04	3.35±0.28
$^{132}\text{Sb}^m$	973.9	99.9±0.1	1.00±0.11	2.91±0.23	0.74±0.12
^{132}Sb	973.9	99.9±0.1			
^{132}Te	228.3	88.2±0.2	4.57±0.36	5.16±0.10	4.50±0.41
^{132}I	667.8	98.7±0.1	4.47±0.34	5.42±0.08	4.34±0.38
^{133}I	529.9	87.3±0.2	5.38±0.39	6.97±0.14	5.17±0.44
$^{134}\text{I}^m$	271.9	79.0±3.0	1.82±0.17	1.16±0.37	1.91±0.19
^{135}I	1131.5	22.8±0.5	4.13±0.35	6.41±0.18	3.83±0.40
^{135}I	1260.4	29.0±0.4	4.20±0.35	6.41±0.18	3.90±0.40
$^{135}\text{Xe}^m$	526.6	81.0±1.0	0.84±0.07	1.71±0.55	0.72±0.06
^{135}Xe	249.9	89.9±0.3 ^c	6.14±0.68	7.60±0.11	5.95±0.76
$^{136}\text{I}^m$	381.4	99.8±5.5	0.90±0.10	1.67±0.13	0.80±0.11

TABLE I. (Continued).

Fission product	E_γ (keV)	Gamma-ray branching ratio (%) ^a	Sample	Cumulative fission-product yield (%)	
				²³⁹ Pu ^b	²⁴³ Cm
¹³⁶ I ^m	1313.0	99.9±0.1 ^f	0.93±0.21	1.67±0.13	0.83±0.24
¹³⁶ I	1313.0	68.0±1.0	0.80±0.19	1.74±0.42	0.68±0.21
¹³⁶ Cs	818.5	99.7±0.1 ^c	0.39±0.04	0.09±0.01	0.43±0.05
¹³⁶ Cs	1048.1	79.8±0.9	0.44±0.04	0.09±0.01	0.48±0.05
¹³⁸ Xe	258.4	31.5±1.3	3.54±0.42	5.17±0.07	3.32±0.47
¹³⁸ Xe	1748.3	16.7±0.7	3.75±0.36	5.17±0.07	3.56±0.41
¹³⁹ Xe	218.8	50.0±6.0	2.09±0.32	3.01±0.09	1.97±0.36
¹³⁹ Cs	1283.2	7.3±0.6 ^f	6.20±0.78	5.28±0.42	6.32±0.88
¹³⁹ Ba	165.8	23.8±0.3 ^q	6.69±1.85	5.53±0.22	6.84±2.10
¹⁴⁰ Cs	602.4	55.7±3.5 ^l	2.85±0.36	3.92±0.32	2.71±0.41
¹⁴⁰ Ba	537.6	24.2±0.2	4.96±0.36	5.37±0.11	4.91±0.41
¹⁴⁰ La	487.0	45.9±0.4 ^r	5.17±0.45	5.38±0.11	5.14±0.51
¹⁴⁰ La	1596.6	95.4±0.1	4.99±0.42	5.38±0.11	4.94±0.47
¹⁴¹ Ba	190.3	46.0±3.0	4.88±0.63	5.21±0.31	4.84±0.71
¹⁴² Ba	1204.0 ^s	22.6±3.1 ^s	2.78±0.45	4.58±0.37	2.54±0.50
¹⁴³ Ce	293.3	43.4±2.0	3.98±0.34	4.35±0.06	3.93±0.39
¹⁴⁴ La	397.4	90.0±5.0	3.19±0.34	3.62±0.29	3.13±0.39
¹⁴⁵ Ce	724.3	63.9±3.9 ^t	3.21±0.37	2.99±0.06	3.24±0.42
¹⁴⁶ Ce	218.3	20.5±3.2	2.28±0.40	2.45±0.10	2.26±0.45
¹⁴⁷ Nd	531.0	13.1±0.8	1.95±0.19	2.03±0.06	1.94±0.21
¹⁵¹ Pm	340.1	22.3±0.5	1.16±0.09	0.75±0.02	1.21±0.10

^aValues taken from our previous evaluation, Table II of Ref. 2, unless otherwise noted.

^bValues taken from current ENDF/B evaluation, Ref. 17.

^cKocher, Ref. 6.

^dGlendenin *et al.*, Ref. 7.

^eGamma ray due to decay of ⁹⁷Nb^m; branching ratio corrected for the fraction of ⁹⁷Zr decay populating ⁹⁷Nb^m. The branching ratio given in Ref. 2 is incorrect.

^fFrom *Table of Isotopes*, Ref. 8.

^gDickens and Love, Ref. 9.

^hHarmatz, Ref. 10.

ⁱReversed from yields given in Ref. 17 for these isomers.

^jDeduced yield of the parent ¹⁰²Mo.

^kDeduced from data reported by Niizeki *et al.*, Ref. 11.

^lTable VI of Ref. 2.

^mDeduced yield of the parent ¹¹²Pd.

ⁿDeduced from level diagram given in Ref. 8; there is a discrepancy between the tabulated and graphical branching ratio in this reference.

^o $T_{1/2}({}^{130}\text{Sb}^m) = 6.5$ min; $T_{1/2}({}^{130}\text{Sb}) = 40$ min.

^pTable II, Ref. 3.

^qGehrke, Ref. 12.

^rDebertin *et al.*, Ref. 13.

^sSum data for nearly degenerate doublet. Relative branching ratios from Ref. 8, combined with absolute branching ratio for $E_\gamma = 255$ keV from Ref. 2.

^tBranching ratio deduced by Reus and Westmeier, Ref. 14. Uncertainty deduced from data of Yamamoto *et al.*, Ref. 15.

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