

## The Thermal Neutron Capture Cross Sections of Pa<sup>231</sup> and Pa<sup>232</sup>

R. ELSON,\* P. A. SELLERS, AND E. R. JOHN  
*Argonne National Laboratory, Lemont, Illinois*

(Received December 3, 1952)

The neutron capture cross section of Pa<sup>231</sup> has been determined to be 293 barns, the largest uncertainty in this determination being due to the uncertainty in the half-life of U<sup>232</sup> and being of the order of 15 percent.

The neutron capture cross section of Pa<sup>232</sup> was determined to be about  $40 \begin{pmatrix} +40 \\ -20 \end{pmatrix}$  barns.

THE cross section of Pa<sup>231</sup> for capture of pile neutrons has been determined by the measurement of the U<sup>232</sup> formed by the beta-decay of the Pa<sup>232</sup> initially formed. Previous values obtained either as above or by direct measurement of the Pa<sup>232</sup> have ranged from 140 to 230 barns.<sup>1</sup>

A two-milligram sample of Pa<sup>231</sup> was irradiated in the Hanford pile along with 4.52 μg of Np<sup>237</sup> as a flux monitor. The amount of Pu<sup>238</sup> formed was determined radiometrically. The total neutron flux was then calculated from the amount of Pu<sup>238</sup> formed and using the following constants: Np<sup>237</sup> half-life,  $2.20 \times 10^6$  years;<sup>2</sup> Pu<sup>238</sup> half-life, 92 years;<sup>3</sup> and the capture cross section of Np<sup>237</sup>, 172 barns.<sup>4</sup> After the sample had cooled sufficiently for essentially all of the Pa<sup>232</sup> to decay (16 half-lives), the sample was dissolved in 10M HNO<sub>3</sub> containing a little HF. An aliquot of this mixture was alpha-counted and pulse analyzed for U<sup>232</sup> at this stage. The fluoride was removed by fuming with perchloric acid, and the uranium was isolated by exhaustive extraction with double volumes of ether from a solution 1M in HNO<sub>3</sub> and saturated with NH<sub>4</sub>NO<sub>3</sub>. There was less than 1 percent by activity of the original U<sup>232</sup> left

in the aqueous solution, as determined by pulse analysis. The uranium was re-extracted into water, and an aliquot was plated and its rate of alpha-emission determined. As pulse analysis showed this sample to be ~95 percent U<sup>232</sup>, the weight could be determined. This value checked that obtained from the assay of the original solution. Assuming that all of the Pa<sup>232</sup> had decayed to U<sup>232</sup> and using a value for the half-life of U<sup>232</sup> of 70 years,<sup>5</sup> a value of the capture cross section for Pa<sup>231</sup> was determined as 293 barns. The largest uncertainty in this determination is due to the uncertainty in the half-life of U<sup>232</sup> and is of the order of 15 percent.

An estimate of the capture cross section for the second-order reaction, Pa<sup>232</sup>(n,γ)Pa<sup>233</sup>, was made. After the uranium was removed from the original dissolver solution, the protactinium in this solution was purified by extraction into di-isopropyl ketone from a solution 6N in HNO<sub>3</sub> and further purified by extraction from a 10N HNO<sub>3</sub> solution with thenoyltrifluoroacetone in benzene.

An attempt to resolve the G-M activity of the protactinium fraction failed, and the amount of Pa<sup>233</sup> present was estimated by following the decay of the sample. As the counting efficiency was not known, this was determined by counting a purified sample of Pa<sup>233</sup> under the same conditions and estimating its weight by following the increase in fissionability due to the U<sup>233</sup> daughter. With due corrections for the counting efficiency, Pa<sup>231</sup> blank, and other long-lived GM activity, an estimate could be made of the Pa<sup>233</sup> formed. Assuming a constant flux during the irradiation, a value is estimated for the Pa<sup>232</sup> capture cross section of

$40 \begin{pmatrix} +40 \\ -20 \end{pmatrix}$  barns.

\* Now at California Research and Development Company, Livermore, California.

<sup>1</sup> Clinton Laboratories Report CL-RWS-20, 1946 (unpublished); A. H. Jaffey and Q. Van Winkle, Argonne National Laboratory, Report ANL-4283, 1949 (unpublished); recalculated from data in a report by Elson, Van Winkle, Bentley, and Ghiorso [Argonne National Laboratory Report CF-3795, 1947 (unpublished)].

<sup>2</sup> L. B. Magnusson and T. J. LaChapelle, J. Am. Chem. Soc. **70**, 34 (1948).

<sup>3</sup> Seaborg, James, and Ghiorso, The Transuranium Elements: Research Papers (McGraw-Hill Book Company, New York, Inc., 1948), p. 226, National Nuclear Energy Series, Plutonium Project Record, Vol. 14B, Div. IV.

<sup>4</sup> A. H. Jaffey and L. B. Magnusson, Argonne National Laboratory Report ANL-4030, 1947 (unpublished).

<sup>5</sup> James, Florin, Hopkins, and Ghiorso, reference 3, Paper No. 22.8.