Neutron-Induced Fission Cross Section of Pu²⁴¹ Below 11 eV*

T. WATANABE AND O. D. SIMPSON

Atomic Energy Division, Phillips Petroleum Company, Idaho Falls, Idaho (Received 5 September 1963)

The cross section for neutron-induced fission of Pu²⁴¹ for neutron energies below 11 eV has been determined. The determination was made using a gas scintillation counter. The multilevel analysis of these data is in agreement with that predicted from the previous analysis of the total cross section.

INTRODUCTION

 $S^{\rm LOW}\xspace$ neutron fission cross-section measurements are of both theoretical and practical interest. In the design of plutonium reactors, this cross section is necessary for the study of long-term reactivity changes and neutron economy. In addition, the analysis of the fission cross section is expected to give added information concerning the fission process. In the analysis of the total cross section of Pu²⁴¹, Simpson and Moore¹ found that the resonances could not be described by the single level Breit-Wigner formula, and attributed this behavior to the fission component. The resonances in the measured fission cross section of Pu²⁴¹ in the 0.1-11 eV energy range have been found to be asymmetric. This has been attributed to interference effects in the present analysis of the fission data as well.

EXPERIMENTAL METHOD

The fission cross section of Pu²⁴¹ was measured using the Materials Testing Reactor (MTR) fast chopper.² The fission events were detected by a gas scintillation detector.³ The detector was divided into four chambers, each viewed by a 2-in. photomultipler tube. The walls of the chamber were coated with sodium salicilate and the

face of the photomultiplier with p-quaterphenyl. Each chamber contained a 2- \times 3-in. foil on which was deposited the fissile material (80.2% Pu²⁴¹ with contaminant of 8.2% Pu²³⁹) by a painting and firing technique. Foils prepared in this manner had an average thickness of 0.1 mg/cm². No check of uniformity was made. The photomultiplier tubes were completely enclosed in the gas volume and electrical connections were made through a multi-lead Kovar seal. The signal was developed across an anode load of 400 Ω and fed directly to a back-biased diode with the bias set to eliminate the alpha particles. The scintillating gas was argon which was allowed to flow continuously through the counter. The number of neutrons per unit time which passed through the fission foil was measured by a pair of BF₃ proportional counters mounted behind the fission chamber. The data were taken at a flight path of 8.5 m at chopper rotor speeds of 400, 1500, and 3000 rpm with resolution of 3 μ sec/m at 0.02 eV and 0.35 μ sec/m at the highest energies. Pulses from the fission chamber and from the BF₃ counters were stored simultaneously in separate sections of a 1024 channel time-of-flight analyzer. An automatic time delay circuit between the fission detector and the BF₃ counter corrected for differences which existed in flight path between them.

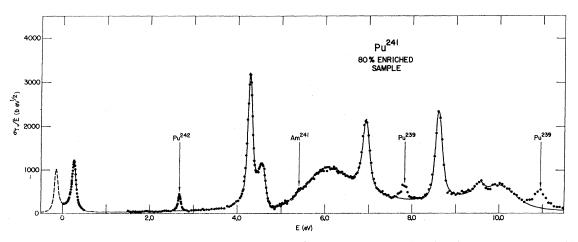


FIG. 1. The total cross section of Pu²⁴¹, multiplied by the square root of the neutron energy, as a function of neutron energy below 11 eV. The solid line shows the multilevel fit to the data.

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³ C. Eggler and C. M. Huddleston, Nucleonics 14, 34 (1956).

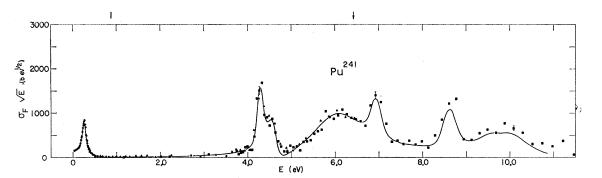


FIG. 2. The fission cross section of Pu²⁴¹, multiplied by the square root of the neutron energy, as a function of neutron energy below 11 eV. The solid line shows the multilevel fit to the data. The circles refer to 400-rpm data, the triangles to 1500-rpm data, and the squares to the 3000-rpm data

DISCUSSION

To convert the relative cross section into absolute cross section, it is necessary to normalize some point in the data to a known absolute value. The normalization of the data was carried out at 6 eV using the total cross section of Pu²⁴¹ as the known value. From the shapes of the total and fission cross section curves shown in Figs. 1 and 2 it is evident that the broad 6-eV resonance is primarily due to fission. From the resonance parameters given by Simpson and Moore,¹ the scattering and radiative capture cross sections of 14 ± 4 and 14 ± 7 b, respectively, were calculated at 6 eV, using the multilevel formula. Subtracting the calculated value of σ_s and σ_c from the measured total cross section of 434 ± 6 b, one obtains a fission cross section of 406 ± 10 b. Since $\Gamma_f \gg \Gamma_{\gamma}$ at 6 eV, uncertainty in Γ_{γ} , and likewise in σ_c , will introduce relatively small error in the derived fission cross section at 6 eV. The measured fission data on Pu²⁴¹ were then normalized to the value of 406 ± 10 b at 6 eV. The normalization of the low energy data was made requiring the integral of $\sigma_f(t)dt$ from 1.5 eV to 100 eV to be equal for the 400-rpm and 3000-rpm runs. Corrections were made below 6 eV for the presence of the Pu²³⁹ contaminant in all runs, subsequent to normalization.

Figure 1 shows the total cross section of Pu²⁴¹, as measured previously by Simpson and Schuman.⁴ Figure 2 shows the fission cross section of Pu²⁴¹, measured as described above. The good agreement of the 1500-rpm data with the other runs tends to verify the normalization technique used to match the high and low resolution runs. The indicated error bars represent only counting statistics. The error in normalizing the low and high resolution data is estimated to be 2.5%. The solid line in both figures represents the multilevel fit to the data up to 11 eV using the parameters obtained by Simpson and Moore.¹ Appropriate resolution broadening corrections have been made. The agreement between the present data and the previous theoretical fit indicates that the assumptions made in the total cross section analysis of Pu²⁴¹ are substantially correct.

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⁴O. D. Simpson and R. P. Schuman, Nucl. Sci. Eng. 11, 111 (1961).